

The Heuristic Drift Model of the Scrape-off Layer: Physics & Implications

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Why Do We Care about the Scrape-Off Layer (SOL) Width?

The width of the scrape-off layer, λ_q , determines the parallel heat flux,

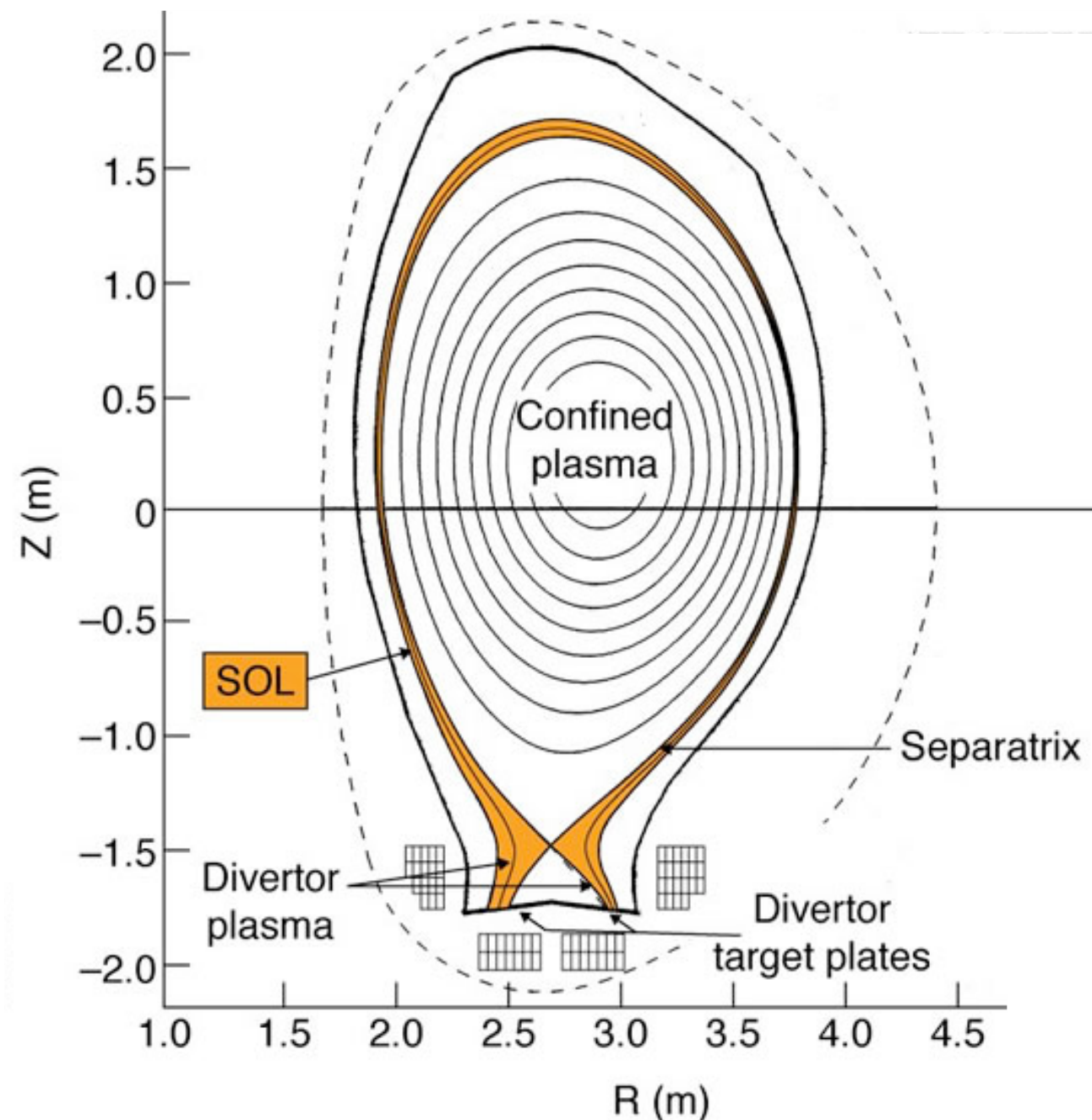
$$q_{\parallel} = \frac{P_{SOL} \left(B/B_p \right)}{4\pi R \lambda_q}$$

which determines the heat flux to the target,

$$q_{\perp, Tar} = q_{\parallel} \sin \alpha$$

and the plasma temperature at the target:

$$q_{\parallel} = \gamma n_{Tar} T_{Tar} c_{s, Tar}$$



Outline

- **Heuristic Drift (HD) Model**
- **Implications for Heat Flux & Impurity Flows**
- **Near-LCFS Heat flux in Limiter Plasmas**
- **Possible Connection to Greenwald Limit**

What Sets the SOL Width?

Standard Analysis for Turbulence Models

Parallel Confinement:

$$\tau_{\parallel} = (qR)^2 / 2\chi_{\parallel} \text{ Conduction}$$

or

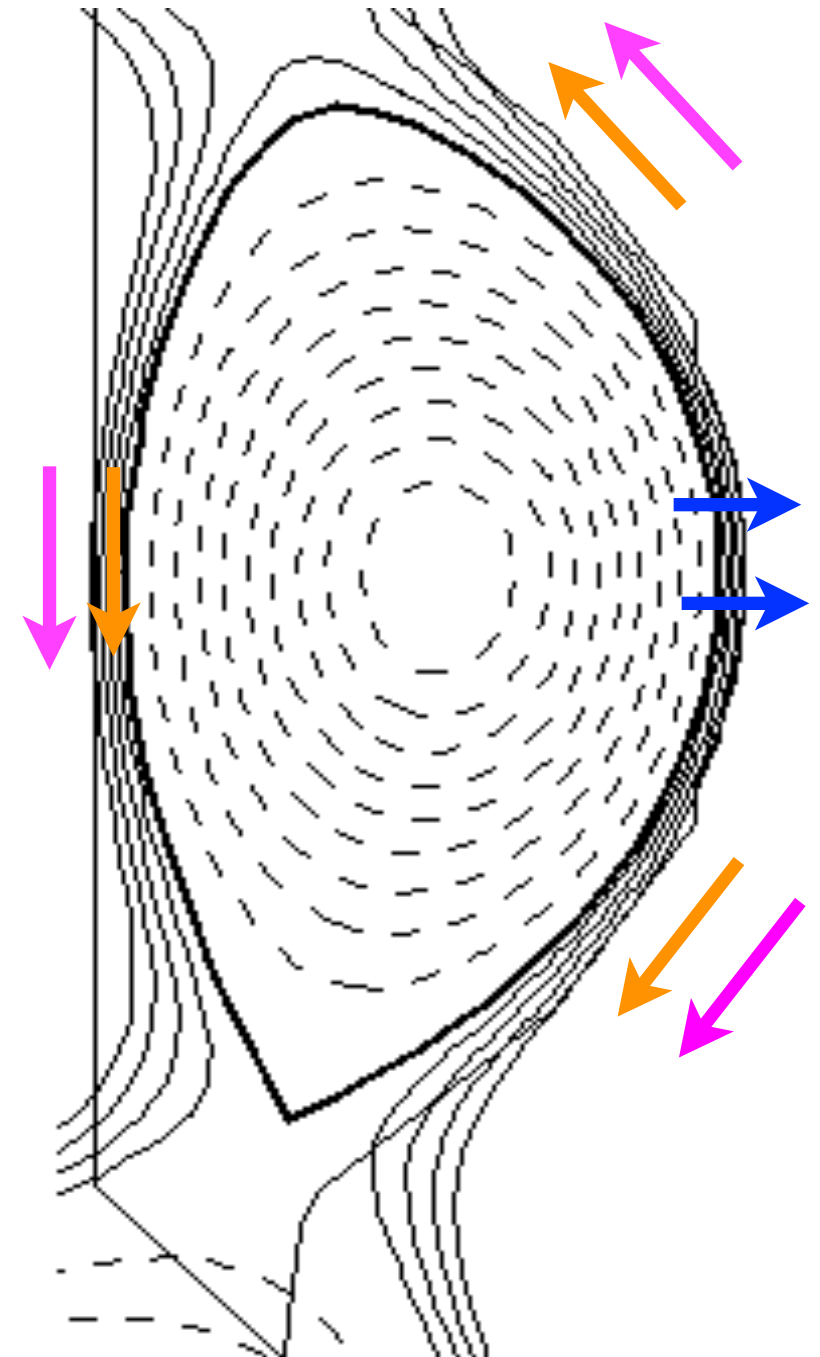
$$\tau_{\parallel} = qR / c_s \text{ Convection}$$

+ Turbulent Cross-Field Diffusion:

$$\begin{aligned} \lambda &= \sqrt{2\chi_{\perp}\tau_{\parallel}} \\ &= \sqrt{\chi_{\perp}(qR)^2 / \chi_{\parallel}} \end{aligned}$$

or

$$\lambda = \sqrt{2\chi_{\perp}qR / c_s}$$



HD Model Width Set by Magnetic Drifts

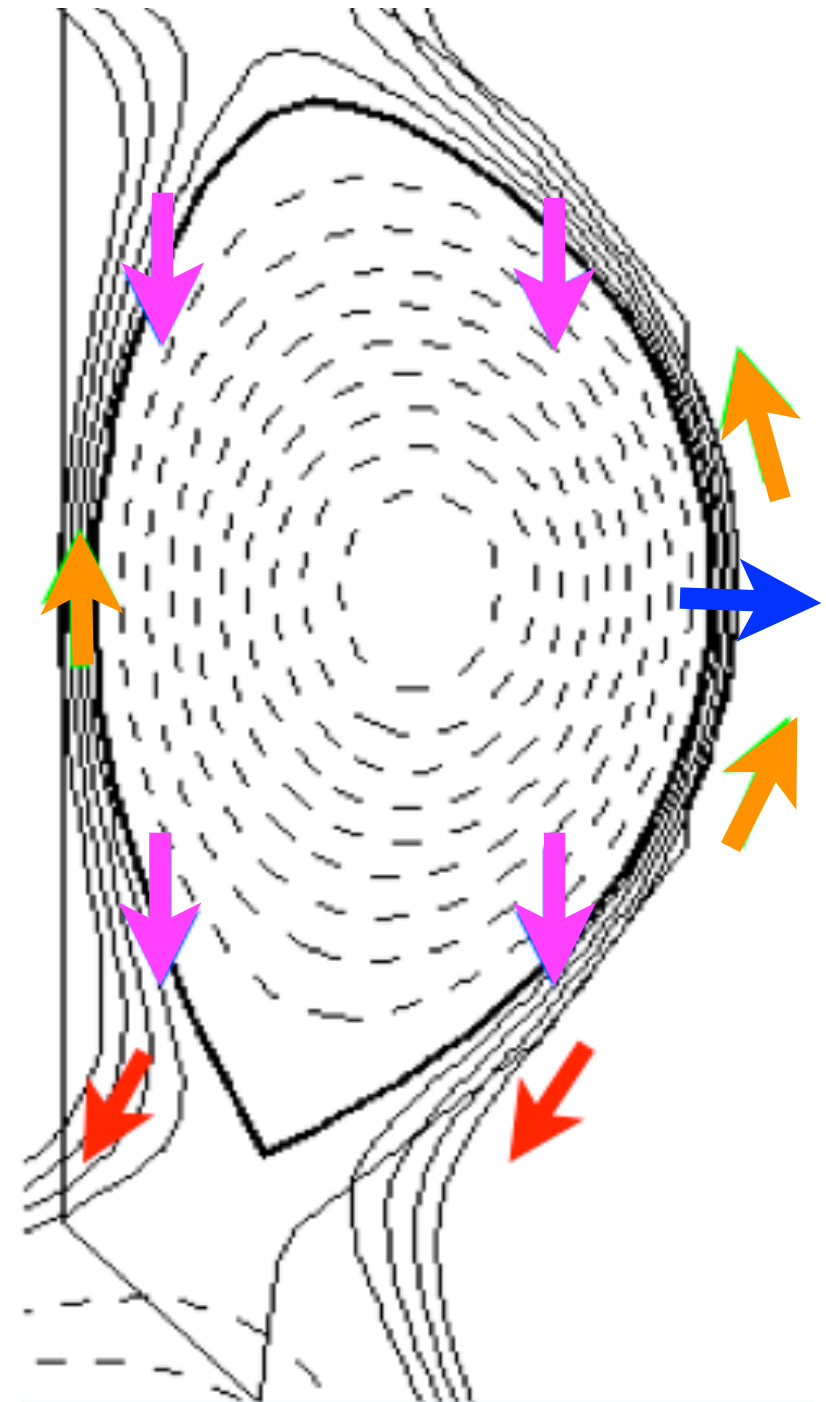
Non-Standard Analysis: $\lambda = v_d \tau_{\parallel}$

- **Vertical drifts cross separatrix into SOL**
- **Parallel flows connect SOL to divertor** at $\sim c_s/2$, consistent with experiments and a simple flow picture, setting a density channel width:

$$\lambda_n \approx v_{\nabla B + \text{curv}B} \tau_{\parallel} \approx 2 v_{\nabla B + \text{curv}B} L_{\parallel} / c_s \approx 2 (a/R) \rho_p$$

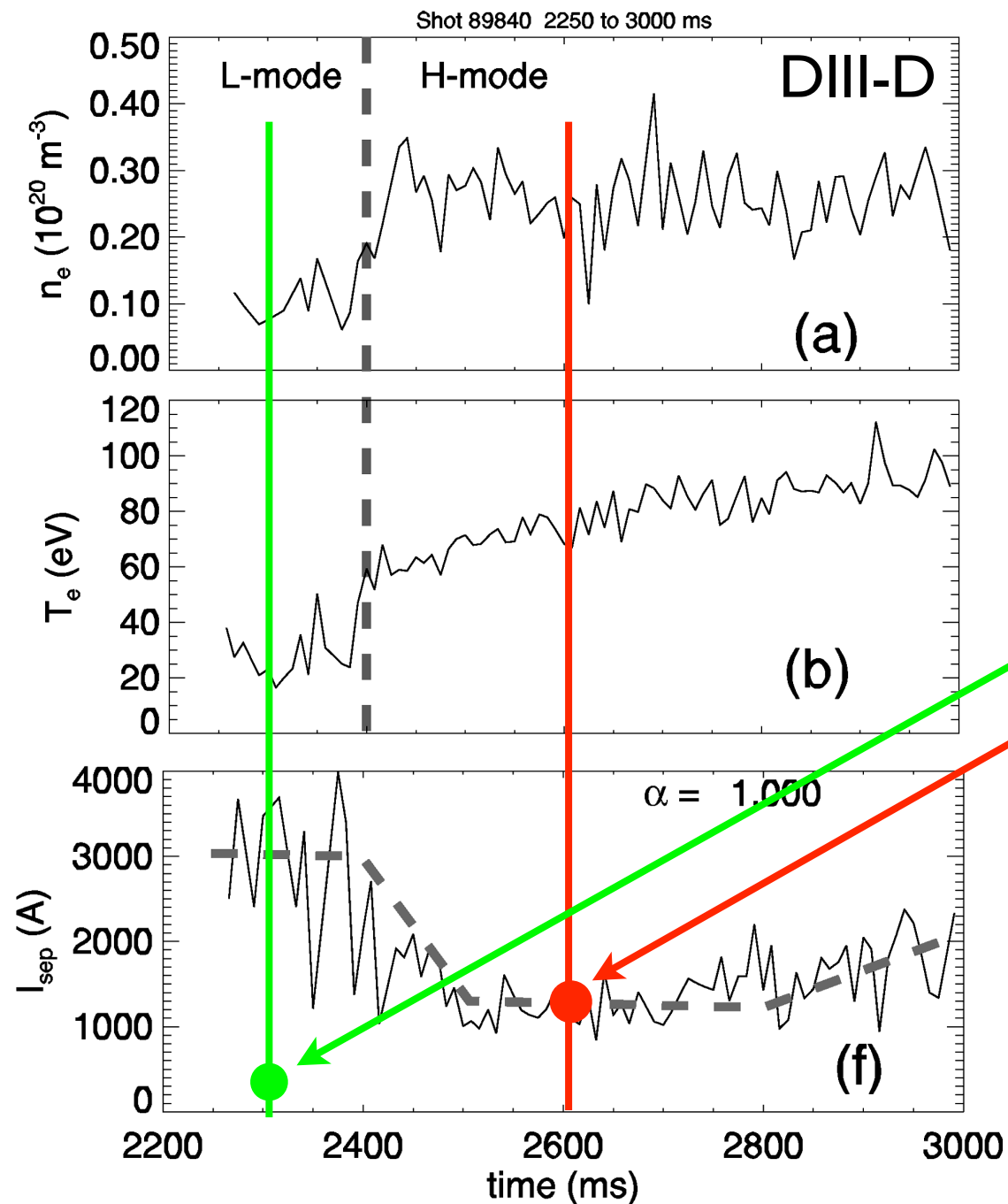
- Implies **Pfirsch-Schlüter flow** also $\sim c_s/2$
- Edge $T_e (= T_i)$ is determined by **anomalous electron thermal conduction** across the separatrix, balanced by Spitzer parallel electron thermal conduction, within λ_n .
- Gives a closed-form, absolute (heuristic) prediction:

$$\lambda_q \simeq 5671 \cdot P_{\text{SOL}}^{1/8} \frac{(1 + \kappa^2)^{5/8} a^{17/8} B^{1/4}}{I_p^{9/8} R} \left(\frac{2\bar{A}}{1 + \bar{Z}} \right)^{7/16} \left(\frac{Z_{\text{eff}} + 4}{5} \right)^{1/8}$$



Measured Particle Loss in DIII-D

H-Mode Equals Drift Loss



Assume that 1/2 of magnetic drift flux returns to plasma via Pfirsch-Schlüter flow, and 1/2 is lost.

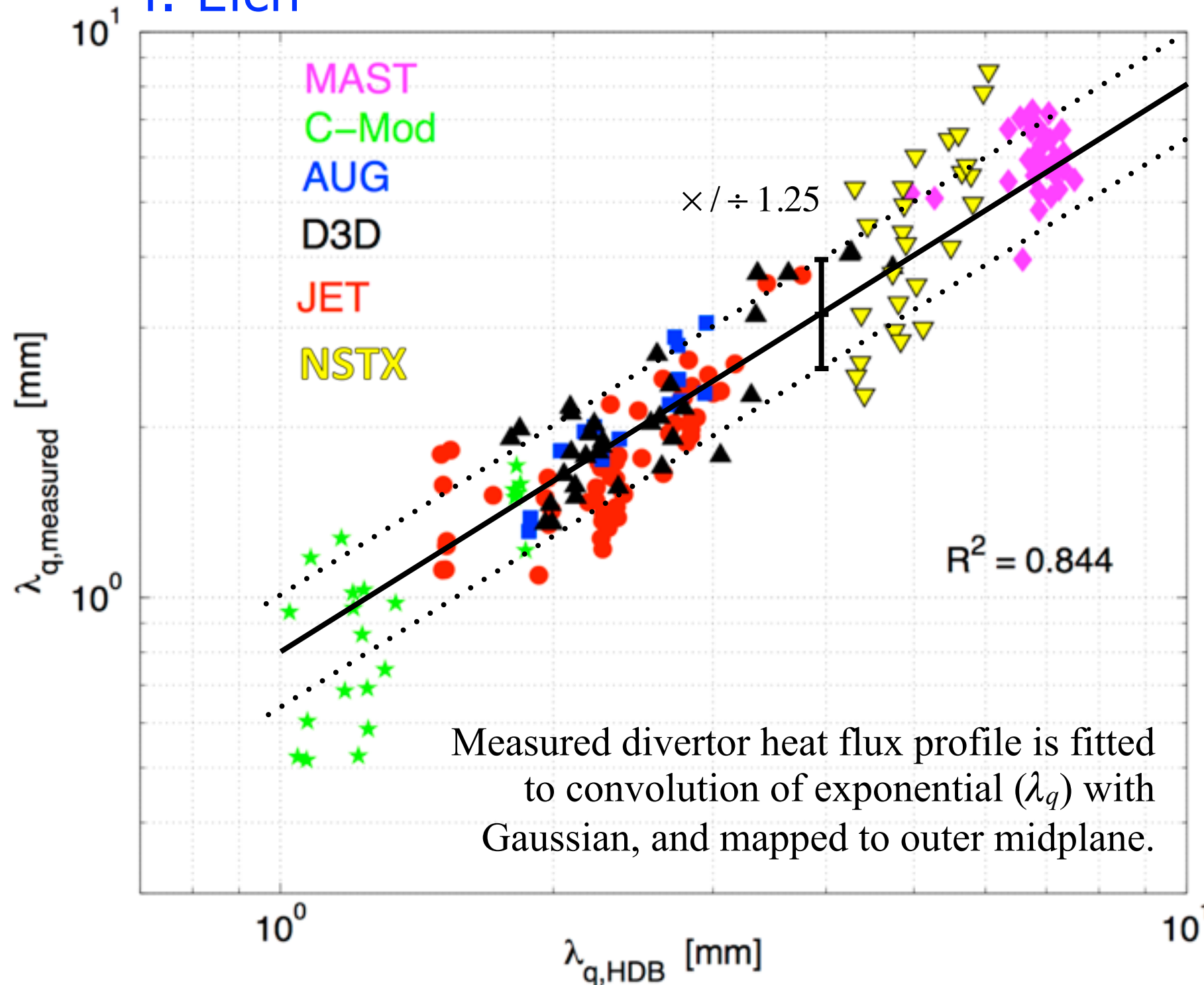
$$I_{loss} = \frac{2n_{sep}T_{sep}}{RB} \frac{2\pi R^2 a}{2}$$

$$\Rightarrow \tau_p = \frac{\pi B R a \kappa n_{core}}{2(T_{sep}/e)n_{sep}}$$

Gives 375 msec for JET,
70 msec for C-Mod.

HD Model Fits Measured λ_q 's Well

T. Eich

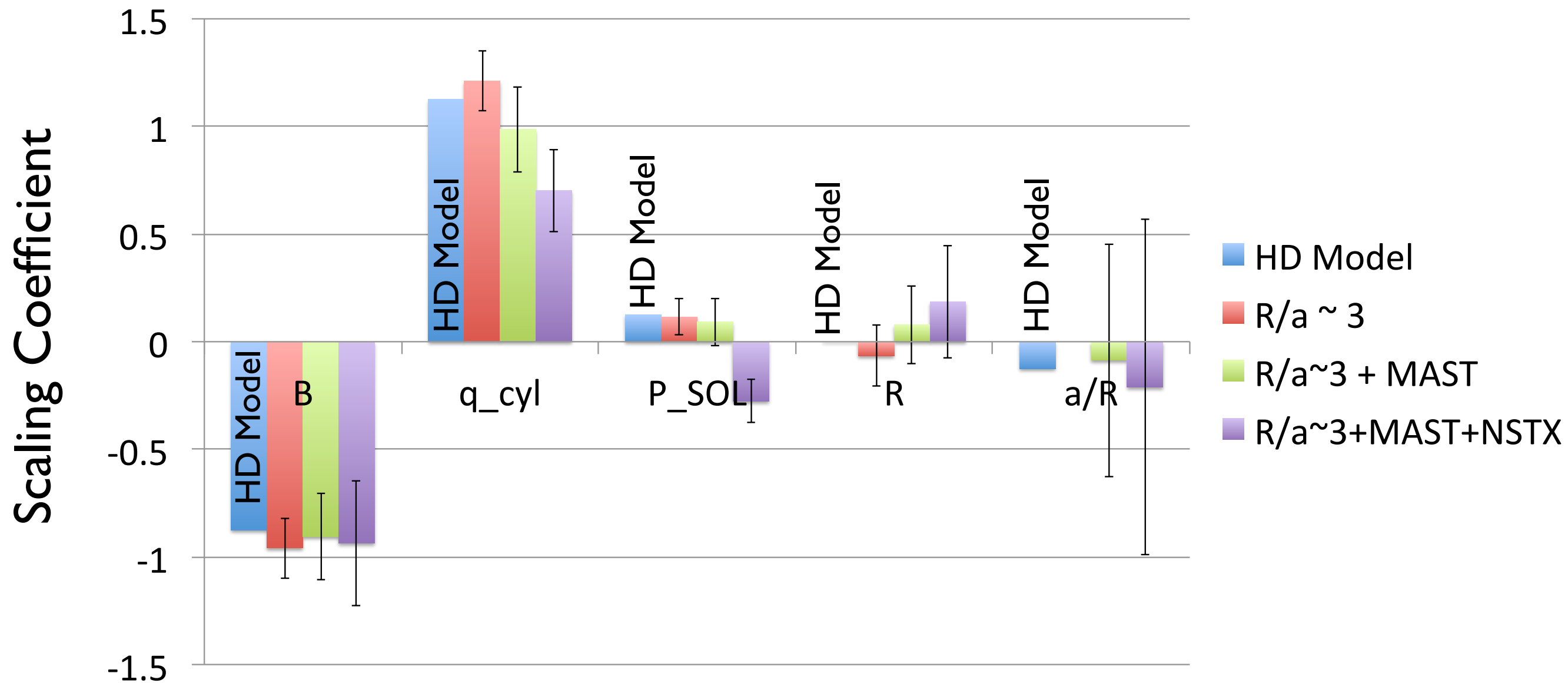


$$\text{Data} \sim 1.6(a/R)\rho_p \\ \sim \text{HD Model} / 1.25$$

Projection to ITER ~ 1 mm
(not counting Gaussian)

Individual Scalings Fit Model

T. Eich



T. Gray

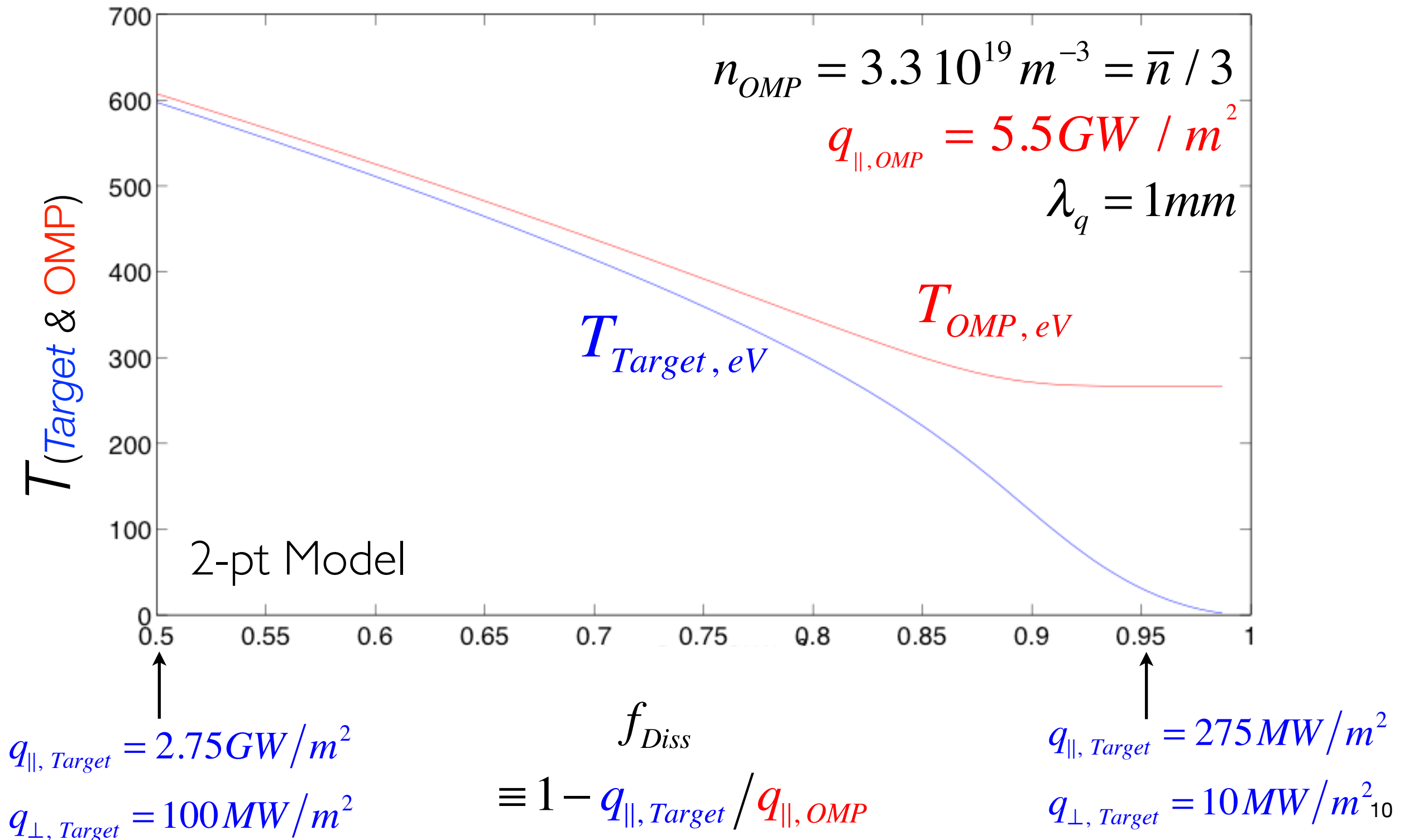
NSTX data set has positively correlated power, current & triangularity.

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- **Possible Connection to Greenwald Limit**
- **Near-LCFS Heat flux in Limiter Plasmas**

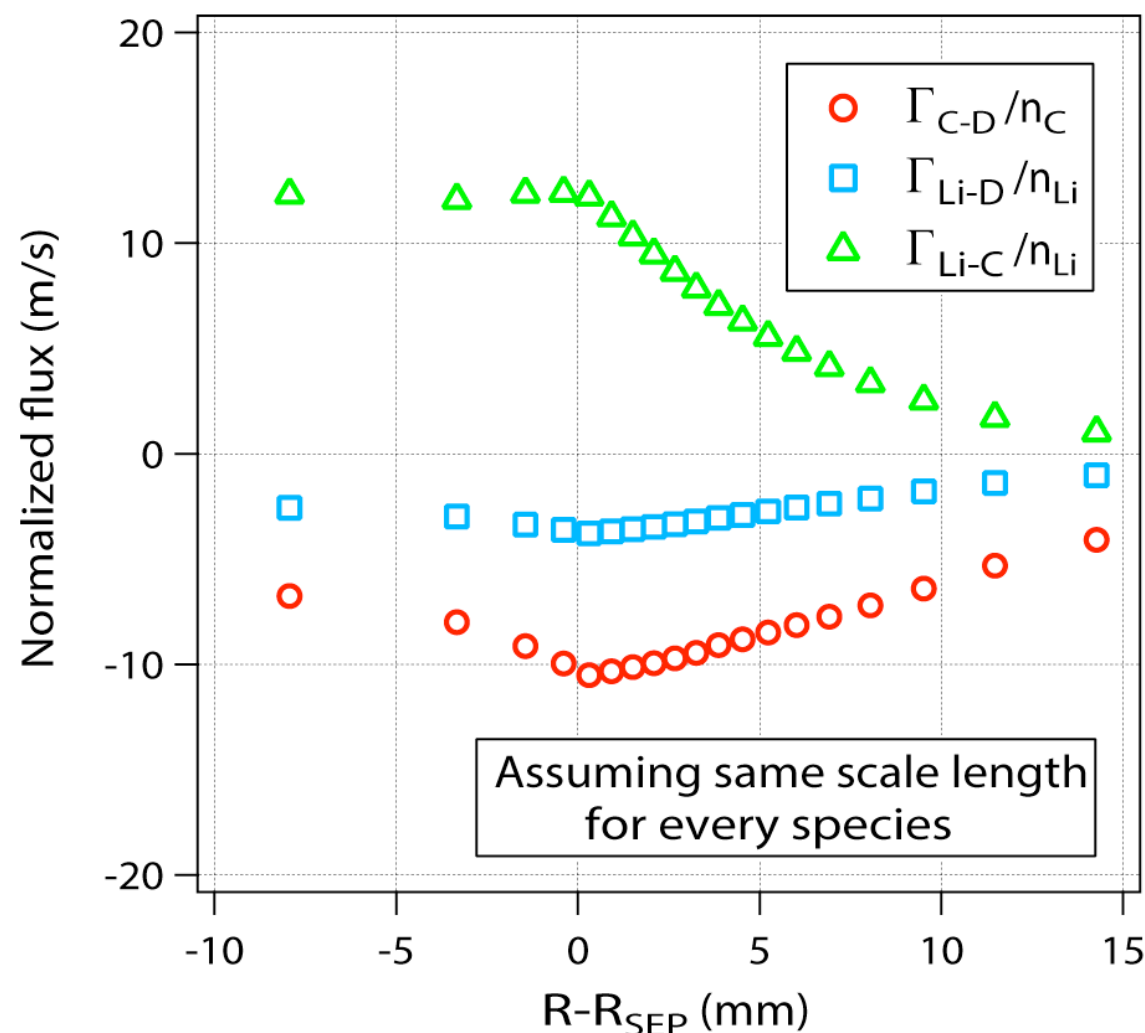
Need High Dissipated Power in ITER

(Transport, Radiation, CX)



Narrow SOL Gives Strong Classical Impurity Flows

- Gradient scale lengths are short, so friction between ion diamagnetic currents is large, causing radial ion exchange.
- High-Z impurities (e.g, C, W) driven strongly inwards for $\lambda_z = \lambda_D$
- Low-Z impurities (e.g., Li) driven weakly inwards by friction with D^+ , outwards by friction with higher-Z for equal λ 's.



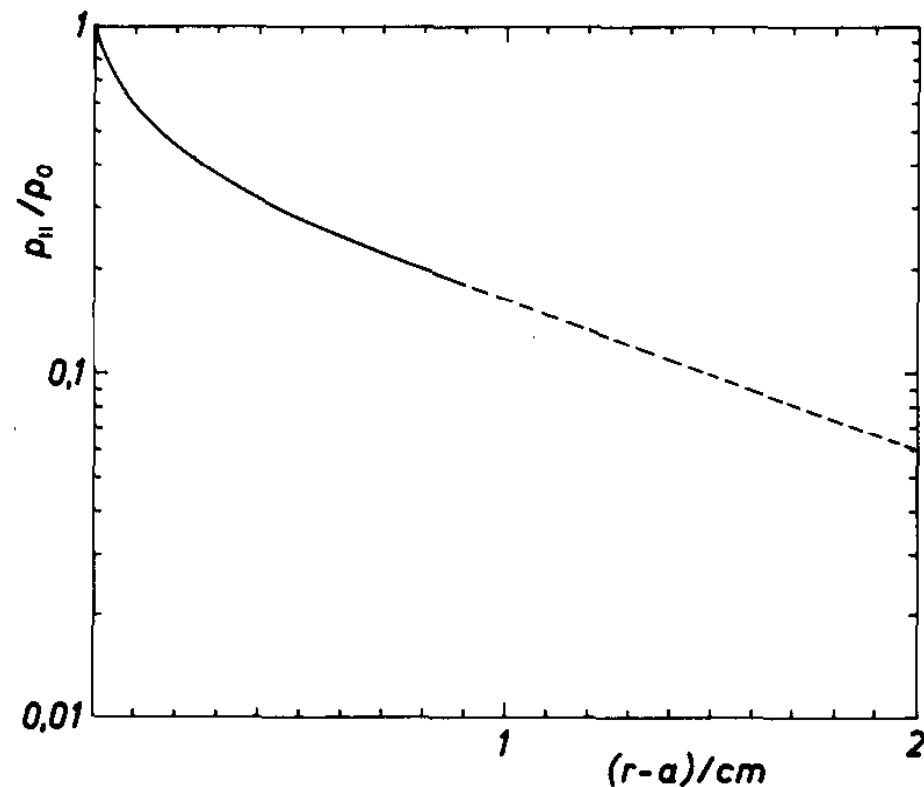
Calculations for NSTX
Outer Midplane Conditions
(no ∇T_i contribution)

F. Scotti

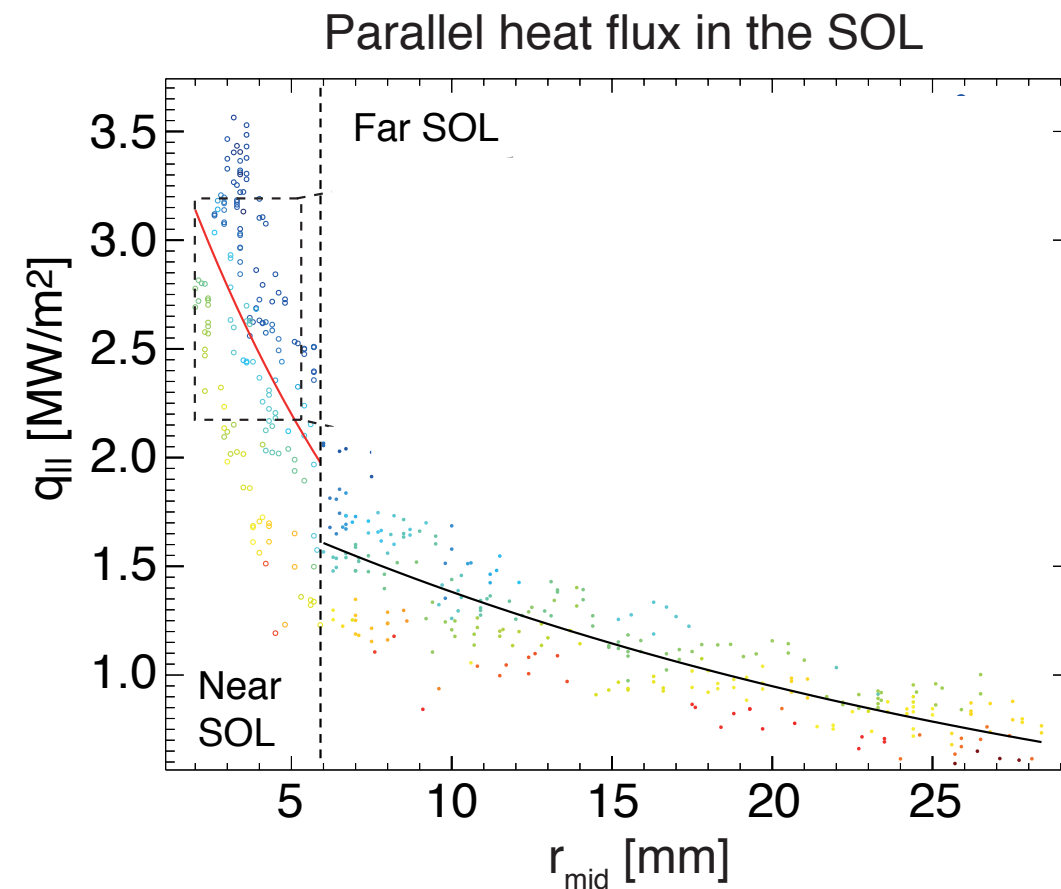
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- **Near-LCFS Heat-flux on Inner-Wall Limiters**
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T-10, TEXTOR and JET Limiter Plasmas Show Narrow Heat Flux Feature near LCFS



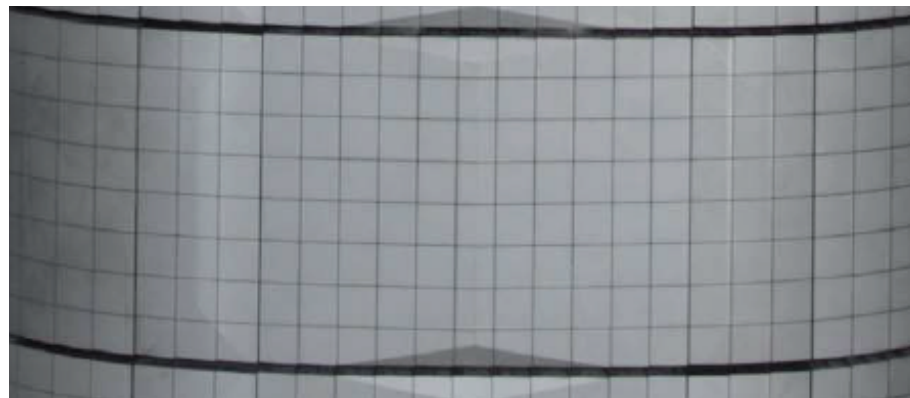
T-10 Pumped Limiter
Chankin, 1987



JET Inner-Wall Limiter
Arnoux, 2013

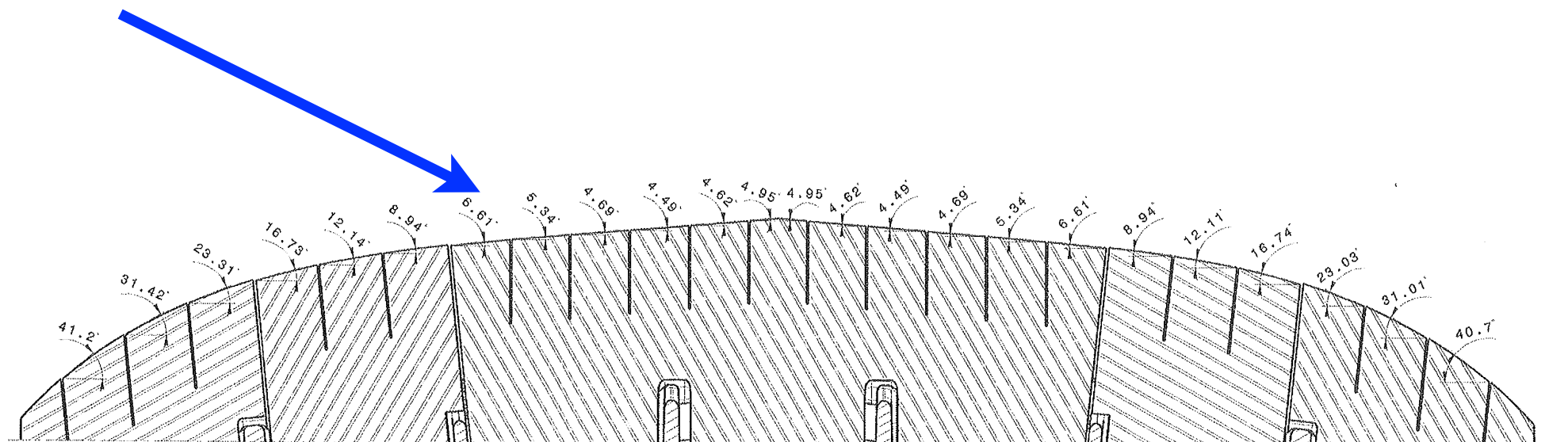
Careful $\sim$ logarithmic shaping to match expected heat flux reveals narrow near-LCFS feature (Not so visible on "unoptimized" rounded limiters!)

The Narrow Feature Resulted in Melting



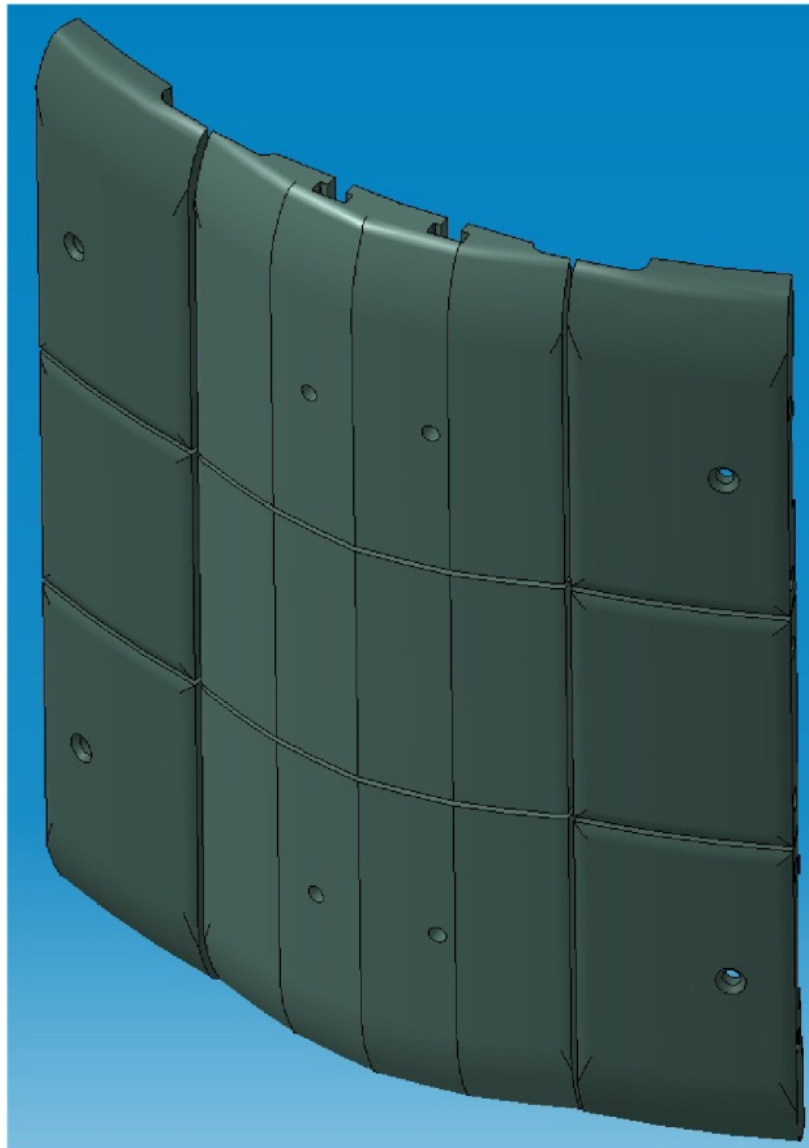
JET Inner Wall Limiter was carefully designed for $1\text{cm } \lambda_q$

Shows melting near the crown



COMPASS Installed a Dedicated Test Limiter

Tiltable between Shots!



Demonstrated that the narrow feature responds to limiter angle $\propto \sin \alpha$

$\Rightarrow$ Not cross-field transport to the limiter surface

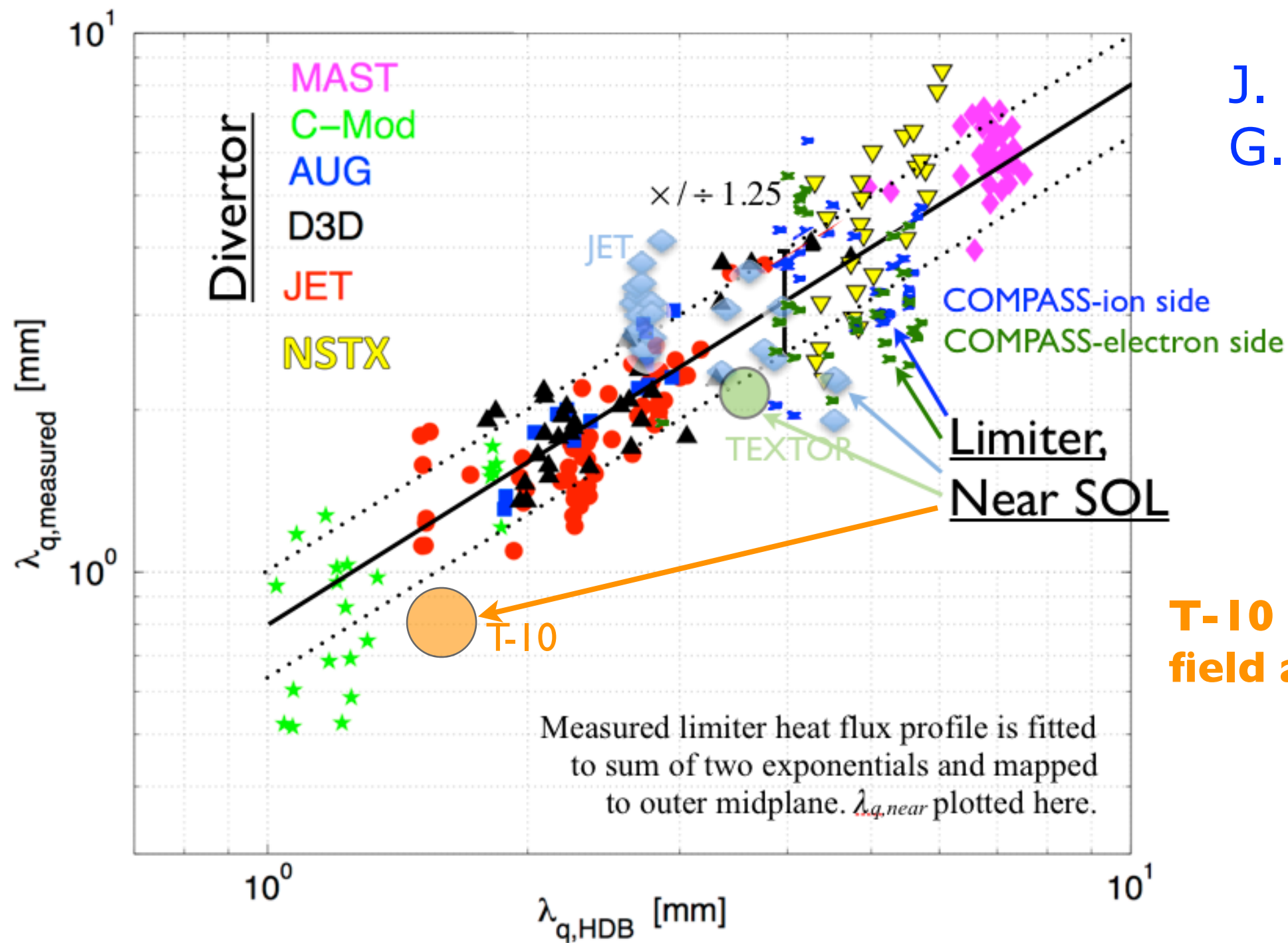
Caveats:

- **Feature not seen on OWL.**
- **Seen sometimes on probes.**
- **Non-ambipolar currents?**

R. Dejarnac, J. Horaček,
M. Komm, R. Panek,
P.C. Stangeby, P. Vondráček

Narrow Limiter Feature $\sim$ HD Model

J. Horaček
G. Arnoux



T-10 data had high field and low $q = 2$.

- Limiters have slightly higher $T_e^{1/2}$ due to sheath-limited $\Rightarrow$ wider than HD
- But also faster flow due to low local re-ionization $\Rightarrow$ narrower than HD

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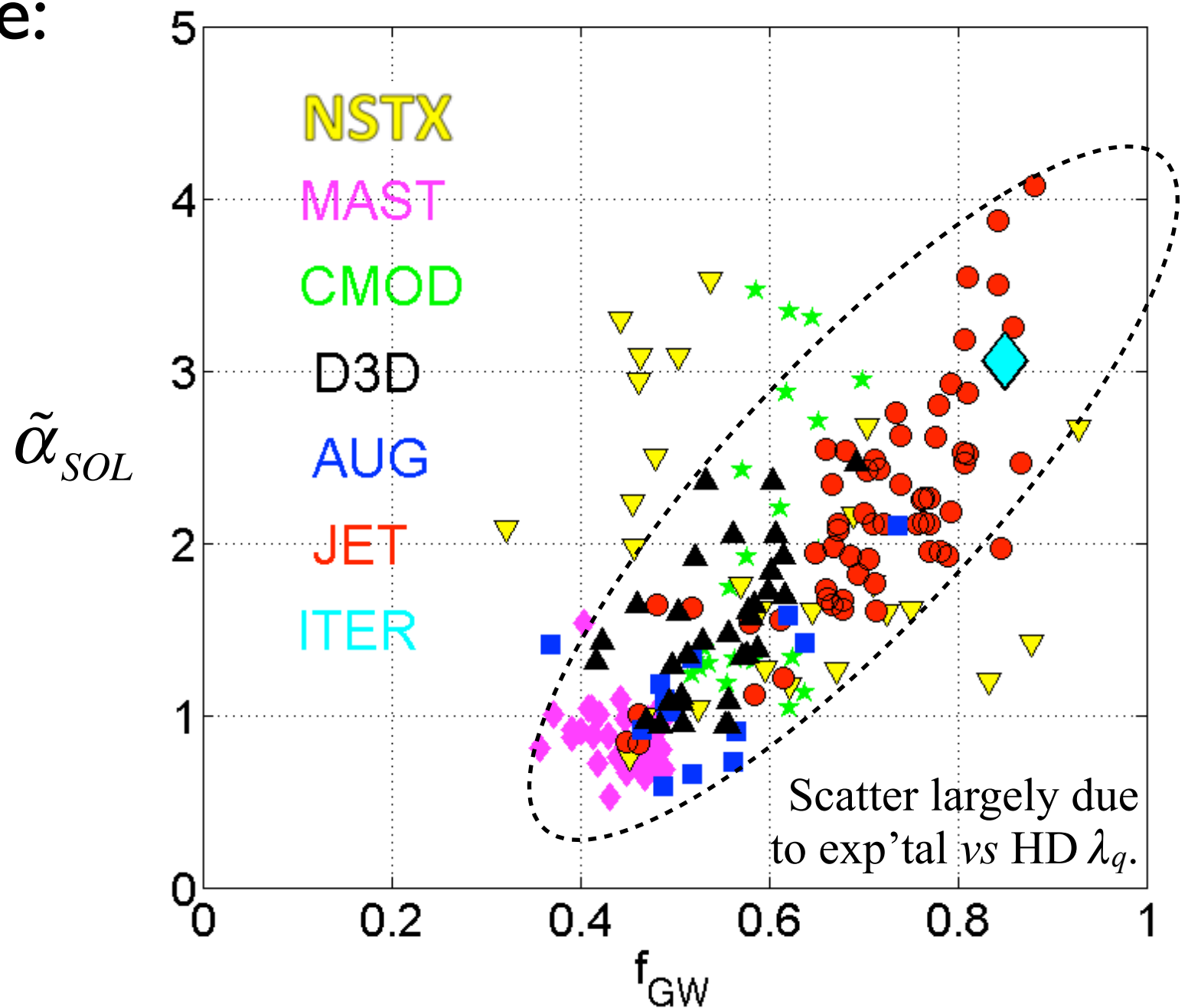
Measured λ_q + Spitzer T_e + $n_{sep} \approx \bar{n}/3$

$\Rightarrow$ MHD Ballooning Drive Rises with n/n_{GW}

MHD ballooning drive:

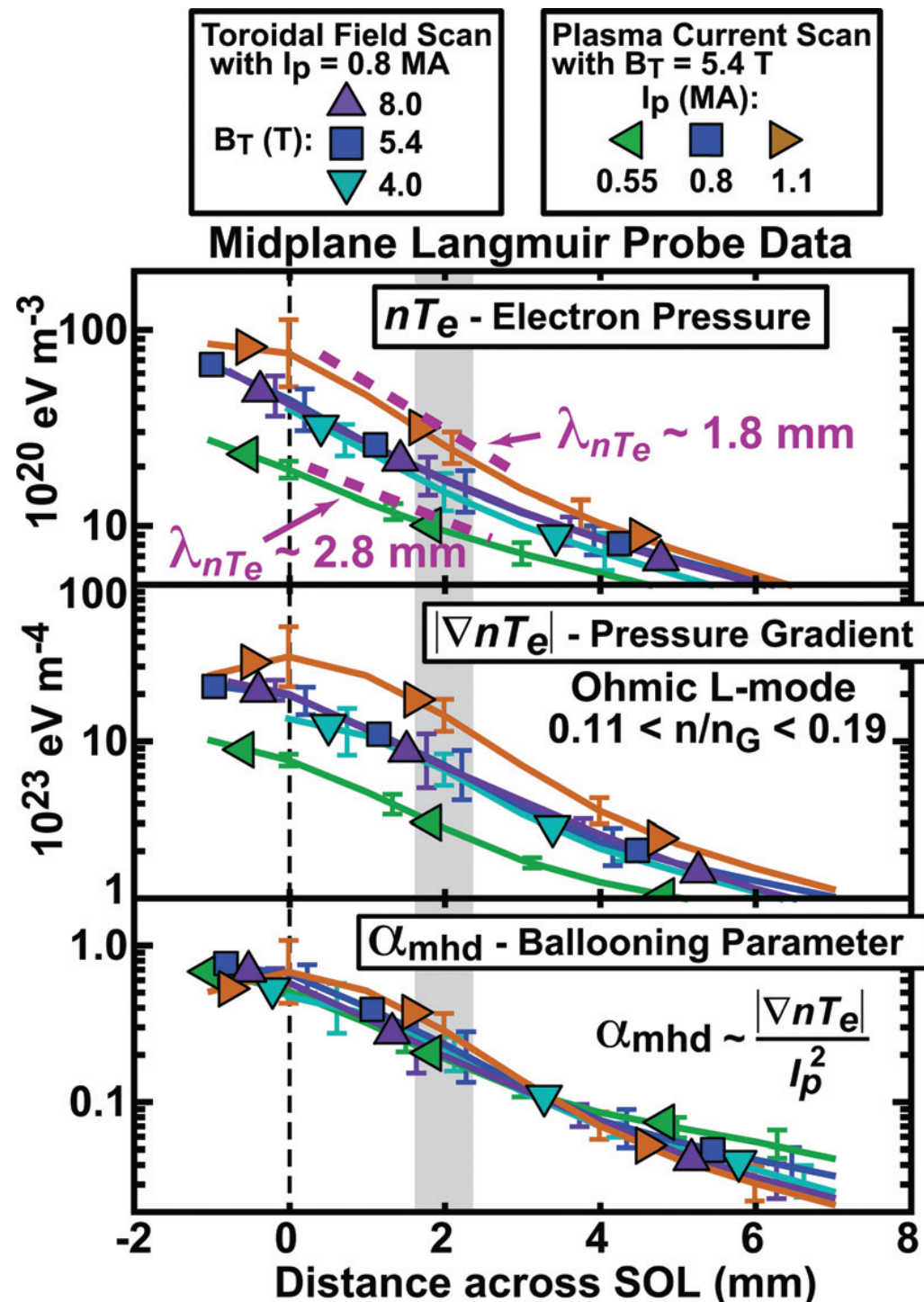
$$\tilde{\alpha}_{SOL} \equiv \frac{Rq_{cyl}^2 2(\bar{n}/3)T_{Spitzer}}{\langle \lambda_{q,exp} \rangle_{pol} (B_t^2 / 2\mu_0)}$$

- Data (and model) give α depending almost exclusively on f_{GW} , without invoking ballooning physics.
- Does the ballooning drive in the SOL kick in at high f_{GW} causing the density limit?



C-MOD Data Show Little Variation in α at Fixed f_{GW} , as Predicted by HD Model

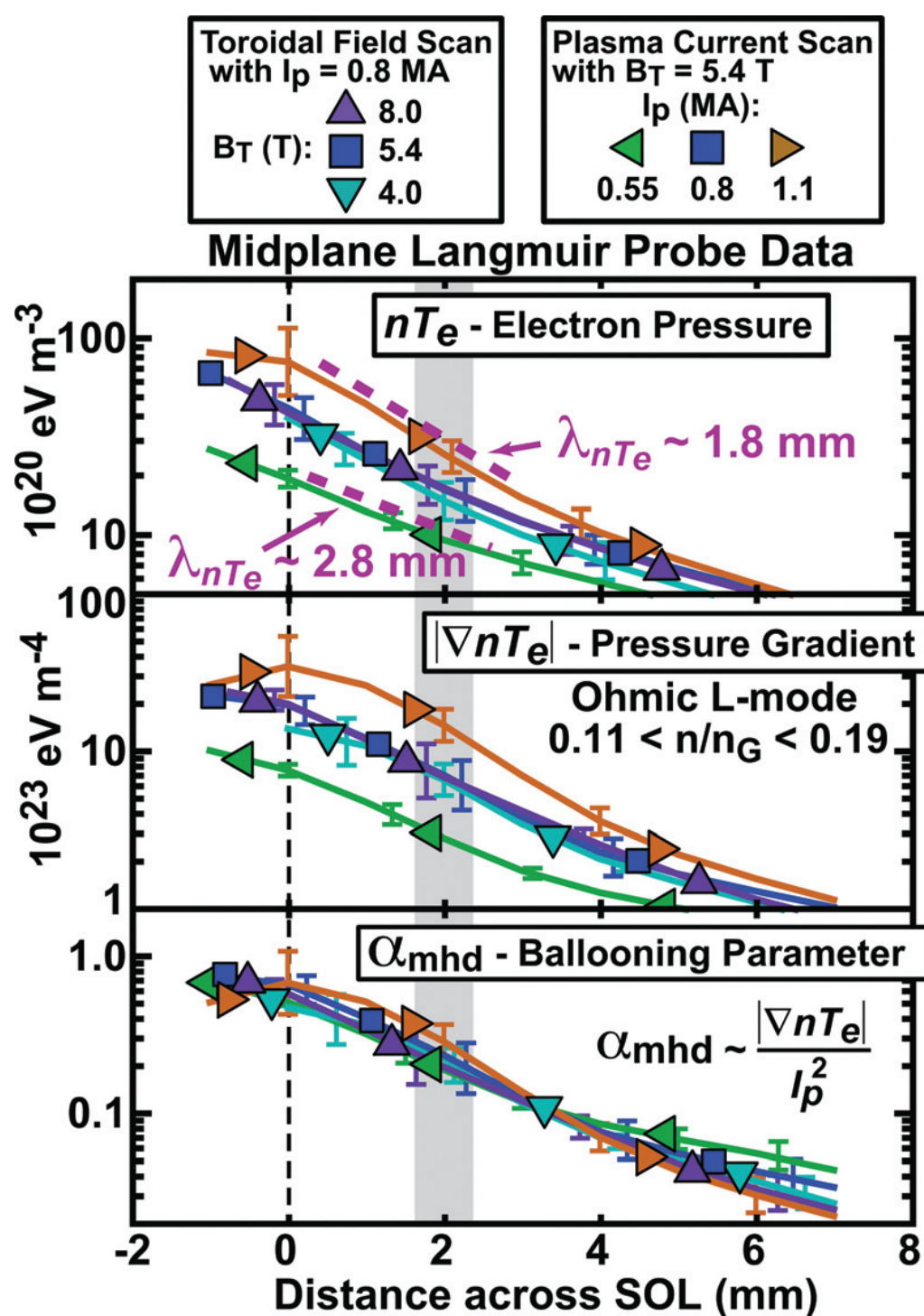
B. LaBombard



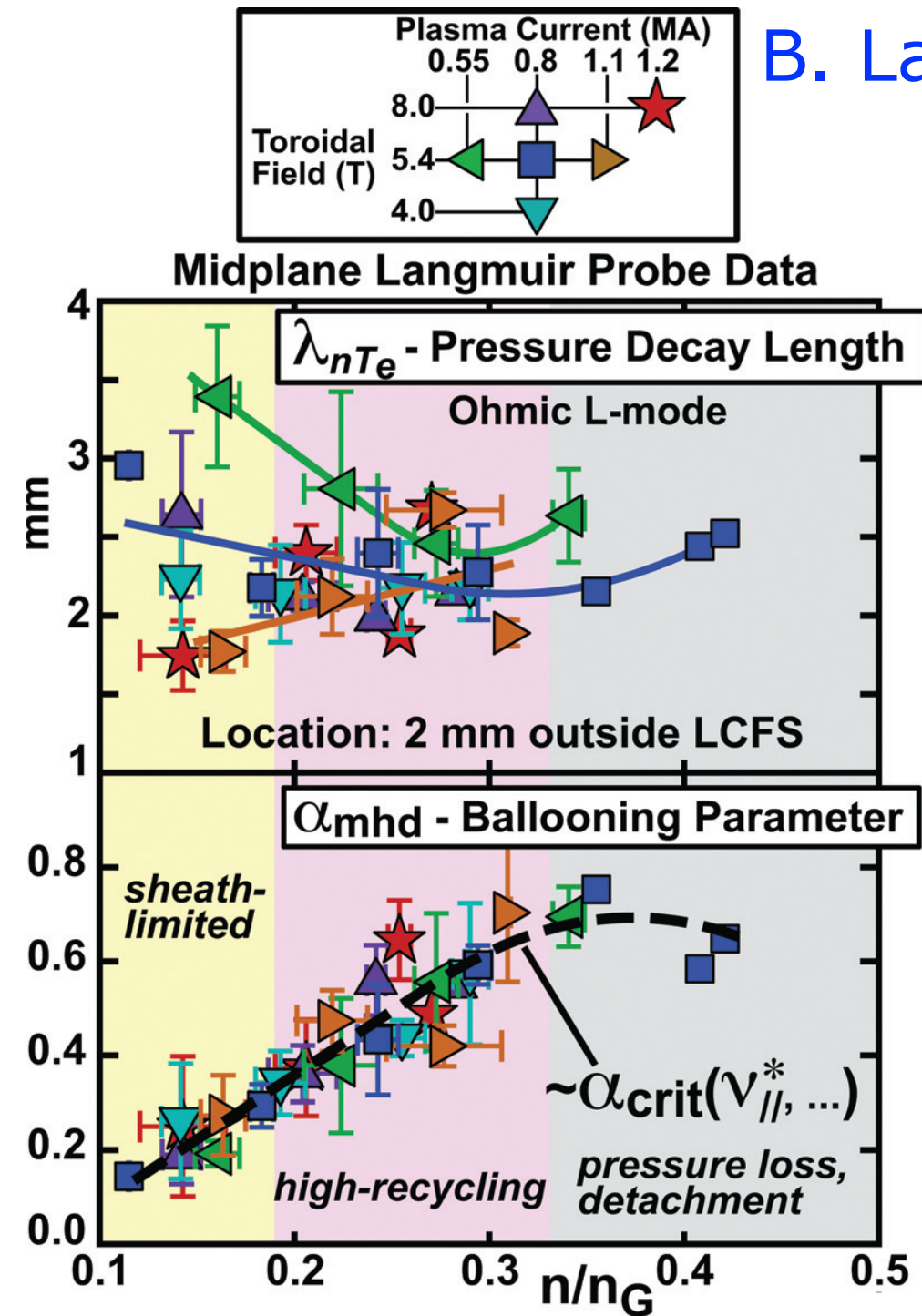
Nearly constant f_{GW}

These Same Data Show α Rises Strongly with f_{GW} – as Predicted by HD Model

B. LaBombard



Nearly constant f_{GW}



f_{GW} Scan

HD Model + MHD Limit

$\Rightarrow f_{GW} \sim 1$ Limit

- Assume $p/p' = \lambda_{q,HD}$, get T from Spitzer
 - Assume $n_{sep} = \bar{n}/3$
 - Assume $\tilde{\alpha} \equiv \frac{Rq_{cyl}^2 (2\bar{n}/3) T_{Spitzer}}{\lambda_{q,HD} (B_t^2 / 2\mu_0)} = \alpha_{crit} = C_\alpha (1 + \kappa^2)^\nu$
 - Solve for $f_{GW,crit} \equiv (\bar{n}/\bar{n}_{GW})_{crit}$
- $$f_{GW,crit} = 24.4 \left[\left(\frac{a}{P_{SOL} B_t q_{cyl} R} \right) \left(\frac{5}{Z_{eff} + 4} \right) \right]^{1/8} \left[\frac{2\bar{A}}{(1 + \bar{Z})} \right]^{9/16} C_\alpha (1 + \kappa^2)^{\nu-3/2}$$
- Numerical value is of order unity
 - Weak dependence on everything but A and κ
 - Myra *et al.* (PP8.38) find $\nu \sim 0.4$ for $n = \infty$ ballooning mode at separatrix, at small $\langle \delta \rangle$. We know δ plays a big role.
 - See also Makowski *et al.* (UP8.36). Need ∇p_i data.

Conclusions

- **A very simple “Heuristic Drift” model is consistent with divertor heat-flux data.**
 - Need to test model in 2-D edge codes, include high flow, determine role of radial, poloidal electric fields.
 - Classical radial impurity flows to be added to 2-D edge codes?
- **Implies need for highly dissipative ITER divertor.**
 - Transport, radiation, charge-exchange.
- **Narrow, high-heat-flux layer on Inner-Wall Limiters $\sim$ consistent with HD model prediction.**
- **Greenwald (and/or H-mode) density limit may be caused by MHD ballooning drive in the SOL.**
 - Need more detailed SOL measurements, especially ∇p_i .
- **Tokamak plasmas seem to be surrounded by $\lambda_q \sim$ magnetic drift speed $\cdot$ poloidal sound transit time. The consequences can be significant.**

Back-up Slides

Field Line Integral gives $\lambda = (2a/R)\rho_p$

Express $\lambda \sim \langle v_d \rangle \tau_{||}$ as the time integral of $\vec{v}_{curv+\nabla B} \cdot \vec{\nabla} \psi_p$ from mid-plane (MP) to x-point.

$$\Delta \psi_p = \int_{MP}^{X-pt} \left(\vec{v}_{curv+\nabla B} \cdot \vec{\nabla} \psi_p \right) \frac{dl_{||}}{c_s/2} = \frac{2}{c_s} \int_{MP}^{X-pt} \left(\vec{v}_{curv+\nabla B} \cdot \vec{\nabla} \psi_p \right) \frac{B}{B_p} dl_p; \quad B_p = \frac{|\vec{\nabla} \psi_p|}{R}$$

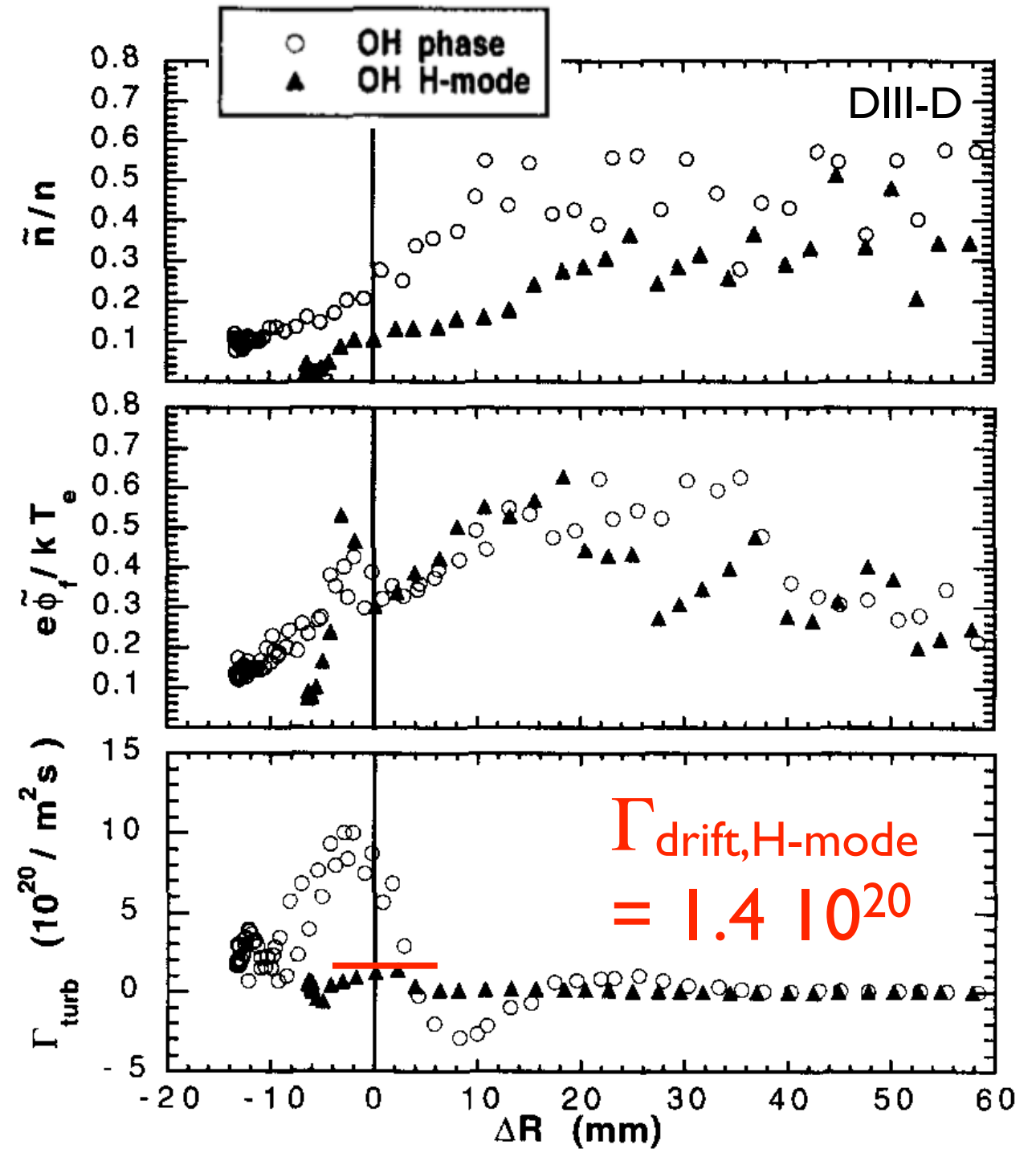
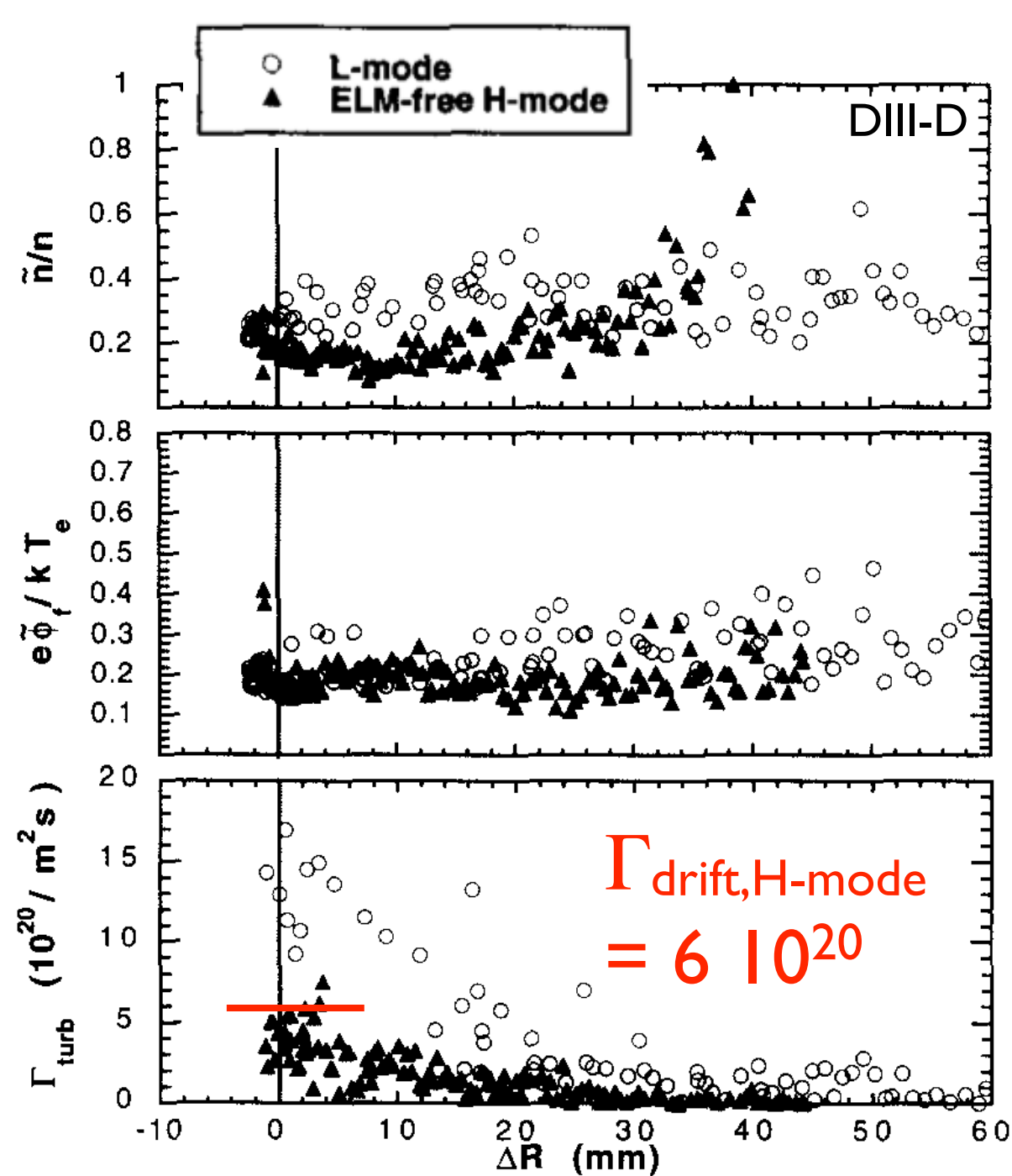
$$\Delta \psi_p = \frac{2}{c_s} \int_{MP}^{X-pt} \left(\vec{v}_{curv+\nabla B} \cdot \frac{\vec{\nabla} \psi_p}{|\vec{\nabla} \psi_p|} \right) RB dl_p = \frac{2}{c_s} \int_{MP}^{X-pt} \frac{2T_{sep}}{\bar{Z}eBR} RB \hat{z} \cdot \hat{\phi} \times d\vec{l}_p = \frac{4T_{sep}}{\bar{Z}ec_s} \int_{MP}^{X-pt} \hat{z} \times \hat{\phi} \cdot d\vec{l}_p = \frac{4T_{sep}a}{\bar{Z}ec_s}$$

$$\lambda = \frac{\Delta \psi_p}{|\nabla \psi_p|} = \frac{4T_{sep}a}{\bar{Z}eB_p R c_s}; \quad c_s = \left[\frac{(1 + \bar{Z})T_{sep}}{\bar{A}m_p} \right]^{1/2}; \quad \bar{Z} \equiv \frac{n_e}{\sum_i n_i}; \quad \bar{A} \equiv \frac{\sum_i n_i A_i}{\sum_i n_i}$$

$$\lambda = \frac{4a}{\bar{Z}eB_p R} \left(\frac{\bar{A}m_p T_{sep}}{(1 + \bar{Z})} \right)^{1/2} \quad \text{or} \quad \frac{4a}{eB_p R} \left(\frac{\bar{A}m_p T_{sep}}{(1 + \bar{Z})} \right)^{1/2} \approx \frac{2a}{R} \rho_p$$

ion drift *electron drift*

In H-Mode: Magnetic Drift Flux $\geq$ Turbulent Flux

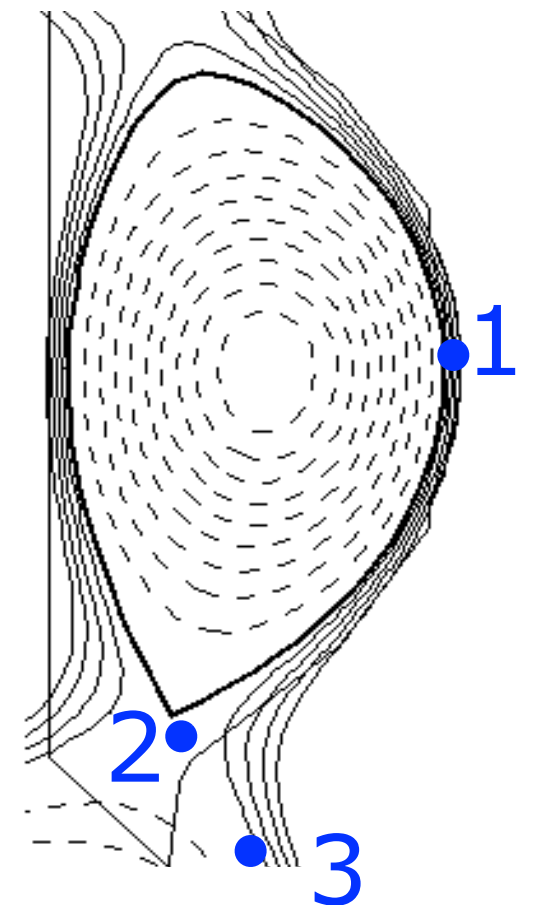
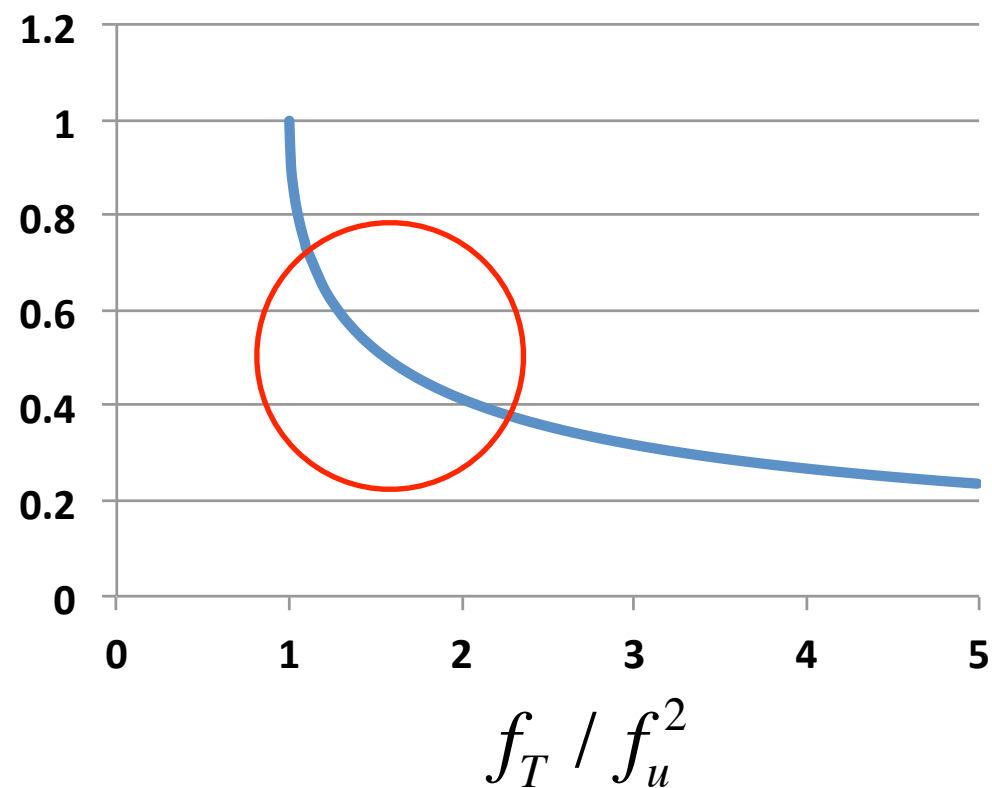
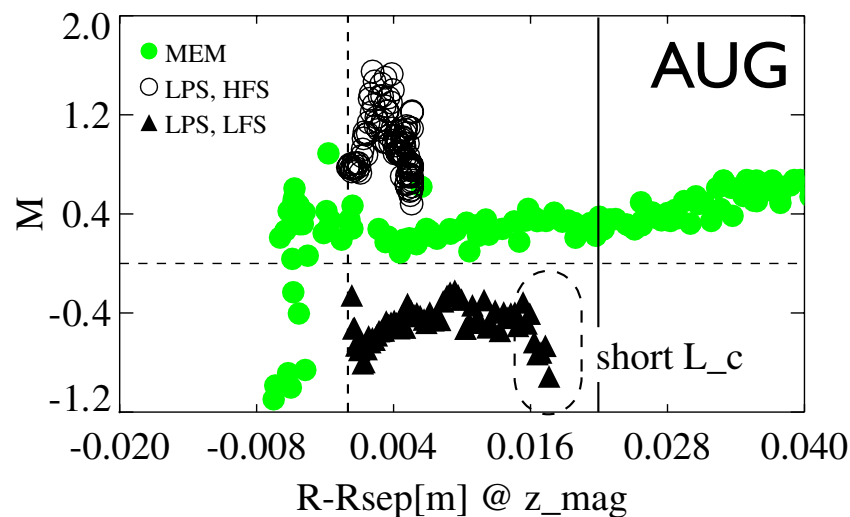


R. Moyer

$C_s/2$ Flow $\sim$ Consistent with 3pt Model

- $f_u \equiv$ upstream source to SOL / total source to SOL $\sim 1/4$
- $f_T \equiv T_{target}/T_{upstream} \sim 1/10$ • $M_x =$ Mach number at X-point

$$M_x = \left(\frac{f_T}{f_u^2} \right)^{1/2} - \left(\frac{f_T}{f_u^2} - 1 \right)^{1/2}$$



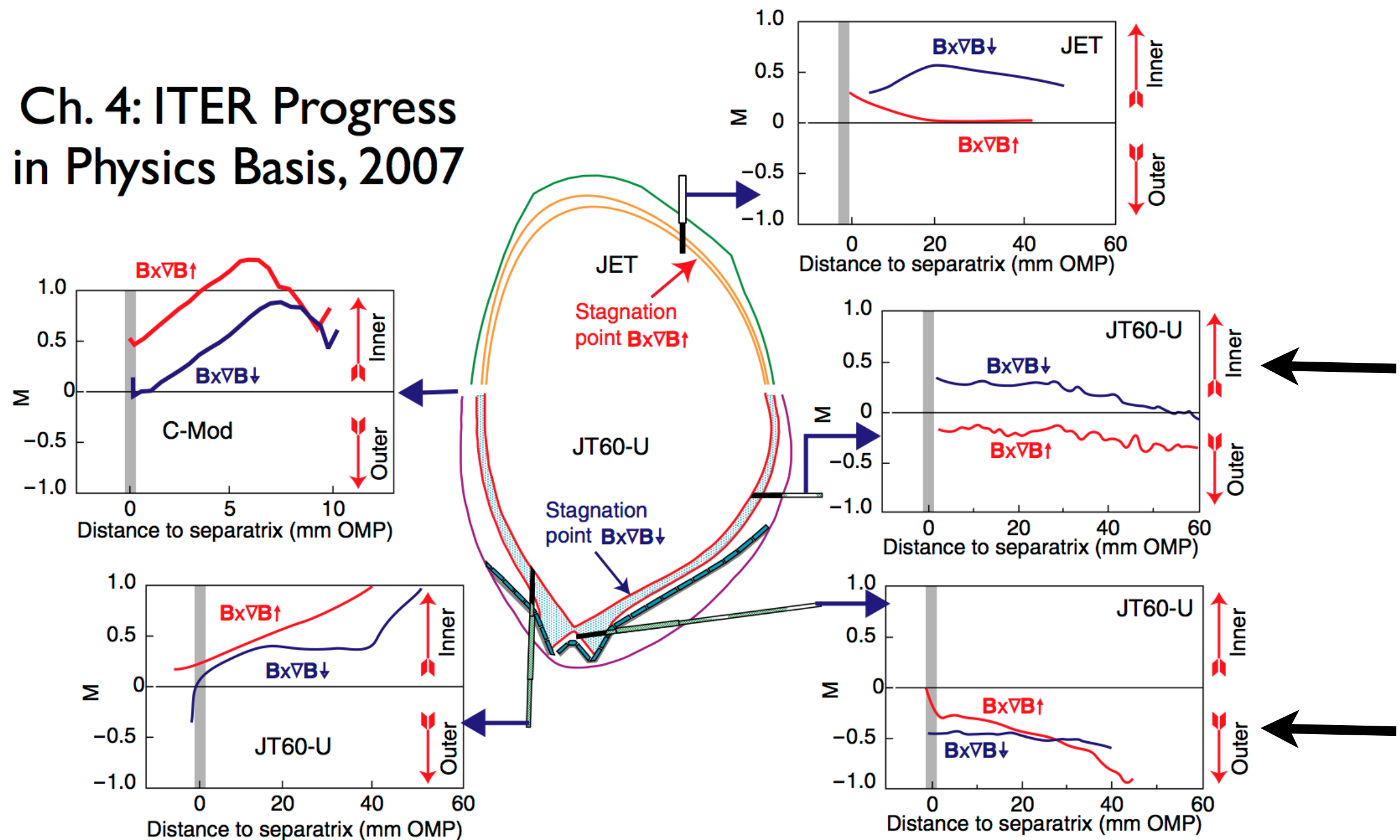
H. Muller

- Lower f_u (better plugging) tends to be tied to lower f_T (colder divertor target) $\Rightarrow M_x \sim$ self-regulating

P. Stangeby

Parallel Flow Picture Supported by Experimental Data

Ch. 4: ITER Progress in Physics Basis, 2007



(Also strong flows in AUG
H-mode near x-point.)

L-Mode data only