

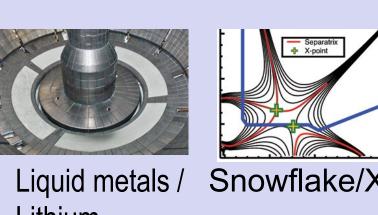
Liquid lithium loop system to solve challenging technology issues for fusion power plant by M. Ono and NSTX-U Team

NSTX-U Mission Elements:

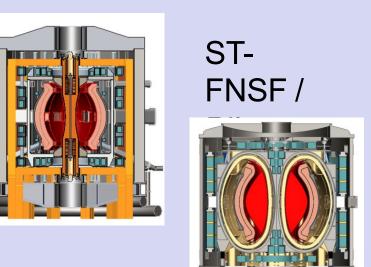
•Explore unique ST parameter regimes to advance predictive capability - for ITER and beyond

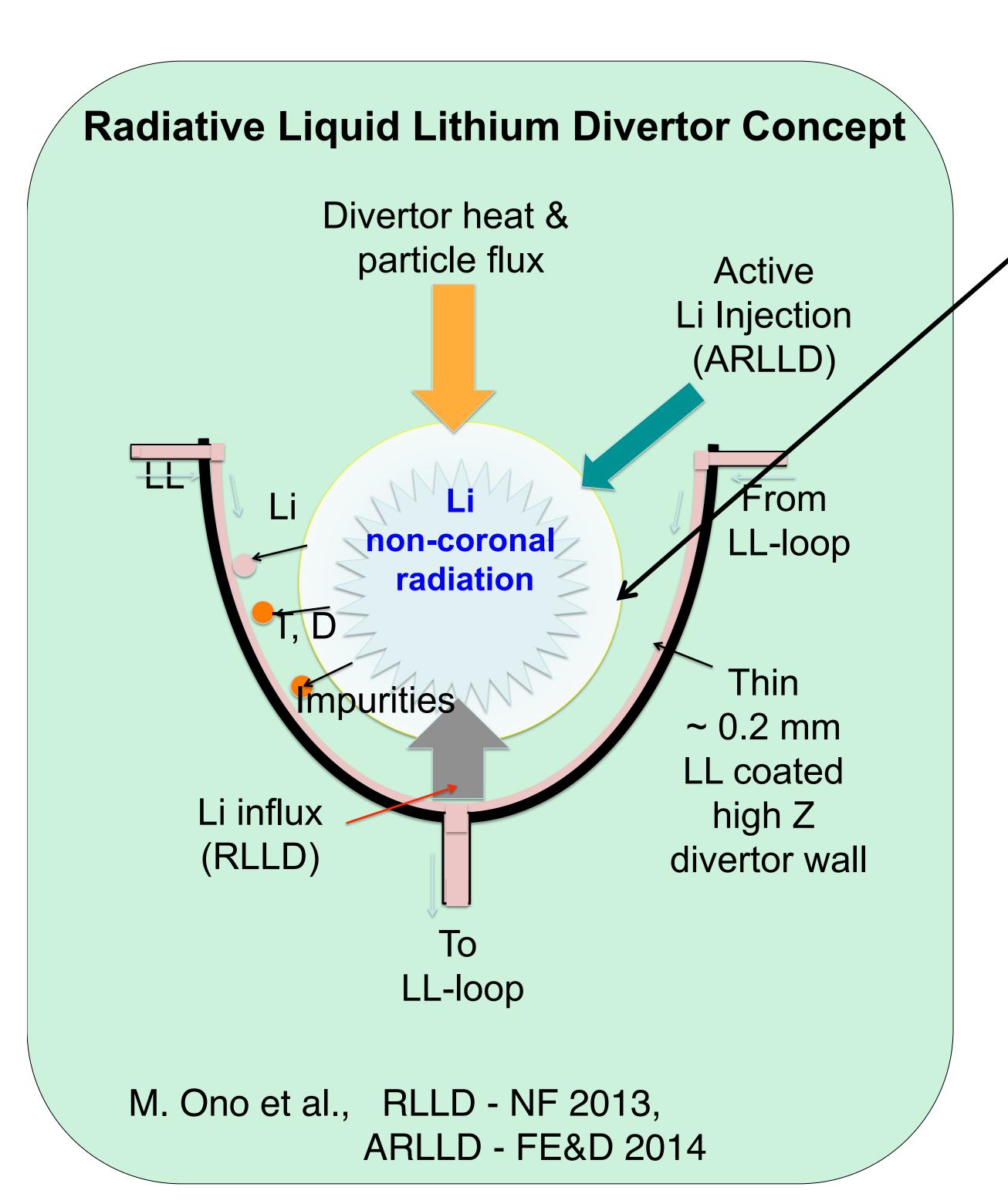


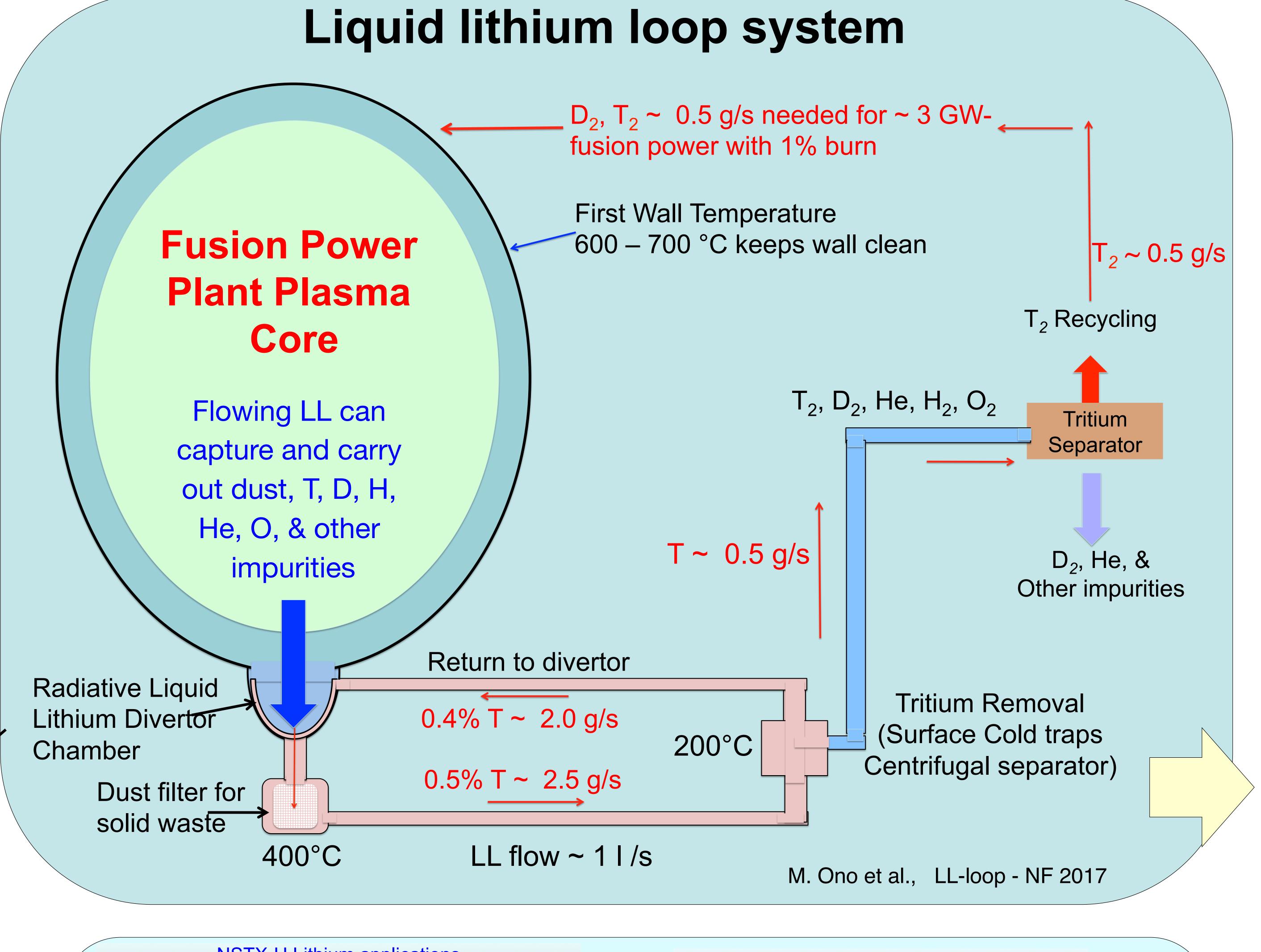
 Develop solutions for plasma- material interface (PMI)

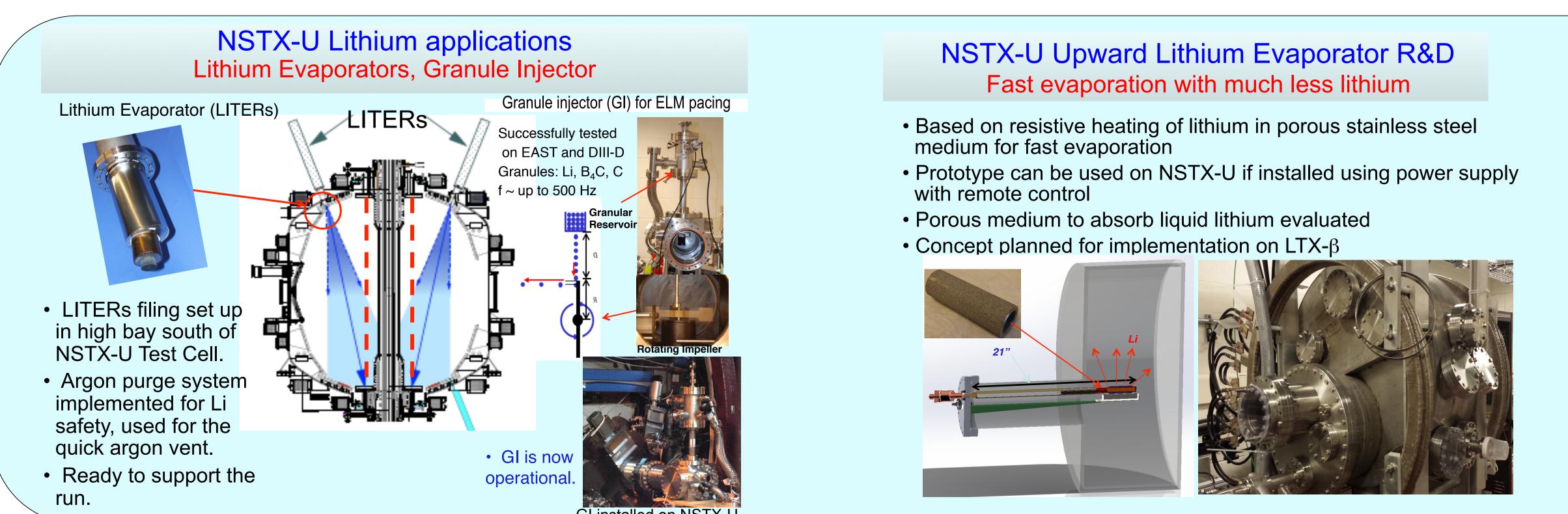


Advance ST as Fusion
Nuclear Science Facility
and Pilot Plant



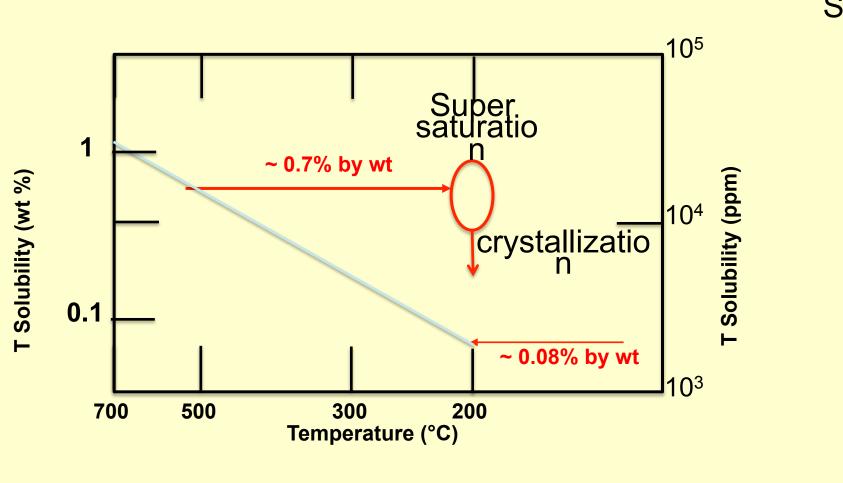


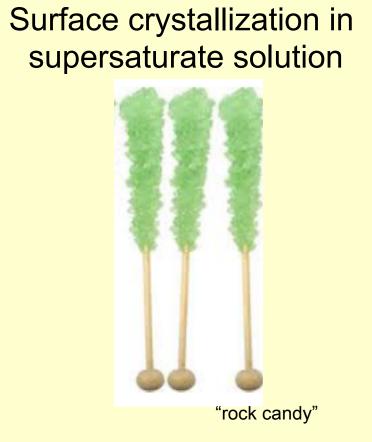




Tritum Removal from Liquid Lithium

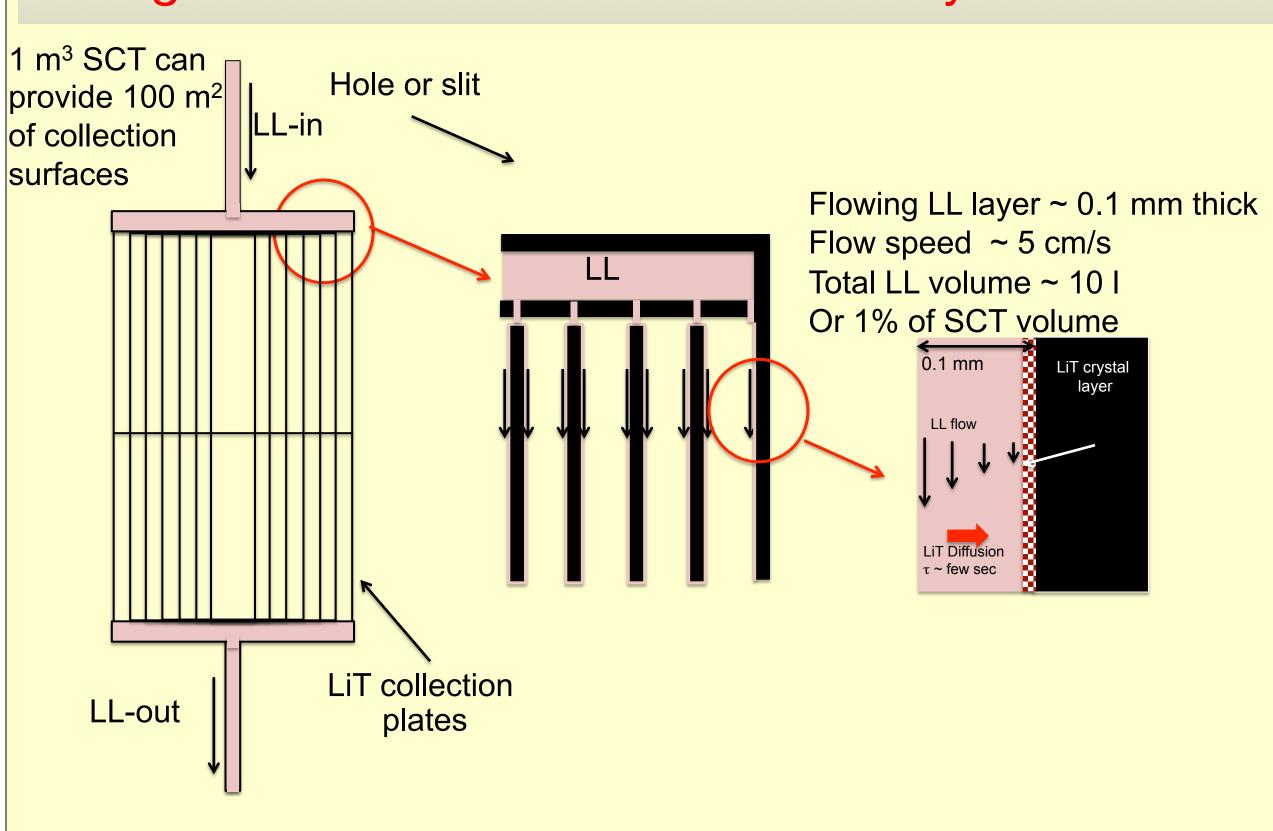
"Cold" Trap at 200 °C Could Remove T, D, H, and O Cold trap can be regenerated at higher temp.





- P.F. Adams JLCM 1975
- At 200 °C, hydrogen can be reduced toward 0.08% level.
- Oxygen is also effectively reduced with cold trap (e.g., IFMIF).
- Nitrogen would require separate hot nitrogen trap if needed.

Surface Cold Trap (SCT) system to provide Large surface area to facilitate LiT crystallization



Centrifugal separation of LiD, LiT appears feasible LiT, LiD ~ 1 g/cc is twice as heavy as LL ~ 0.5 g/cc

