

# Spherical Torus Fusion Development Path

## Realistic Steps Toward Attractive DEMO

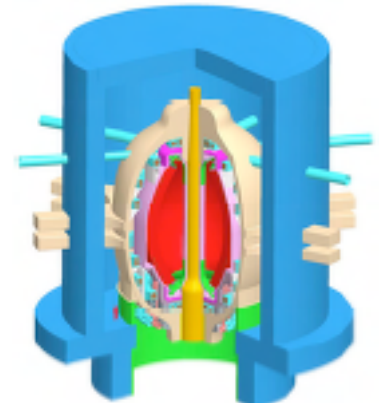


ST is a low-aspect-ratio tokamak ( $R/a \sim 1.5$ )

Attractive physics features:

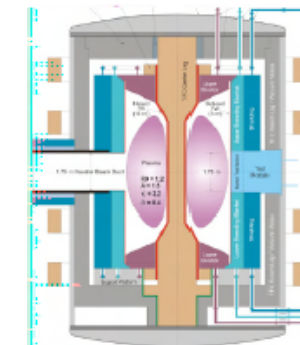
- Naturally high elongation
- High beta
- High shape-factor

ST-DEMO



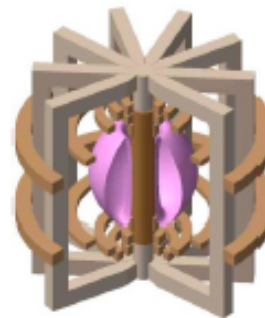
ARIES-ST

ST-CTF



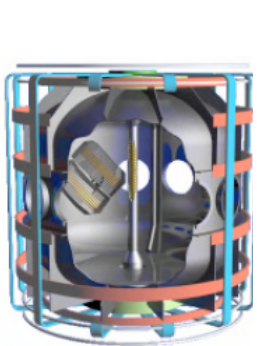
Compact CTF

ST-PE

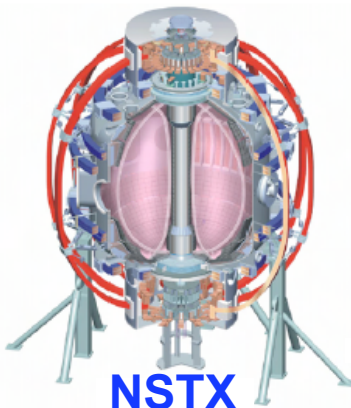


NHTX

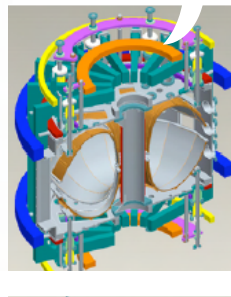
ST-PoP



PEGASUS



NSTX



LTX

Attractive reactor features:

- A simple inexpensive demountable TF Coil.
- Allowing radically simplified maintenance.

# U.S. is Leading the Vibrant World ST Research Program



- Active world wide ST research during the past 10 years in the U.S., U.K., Japan, R.F., Brazil, Italy, and PRC with 12 operating ST experiments and three new facilities are under construction.
- NSTX (US) and MAST (UK) are mega-ampere class ST facilities.
- US ST program is leading the world in science and innovation:

**NSTX** - The most powerful ST facility in the world with state-of-the-art diagnostic systems, addressing the full range of scientific topics, including ST-specific integration issues.

**HIT-II** - Developed CHI to start the plasma non-inductively; Applied on NSTX.

**PEGASUS** - the lowest  $R/a \sim 1.1$  to explore the benefit of this regime including very high beta operations. Also investigating an innovative start-up technique based on plasma guns which can be also applied to NSTX.

**LTX (CDX-U)** - the lowest particle recycling ( $\sim 30\%$ ), and improved confinement with liquid lithium based approach. To be applied on NSTX.

**ST is a part of the world fusion research portfolio of concepts to maximize the chance of successful DEMO**

# International ST Research Activities



- **Operating International STs:**

**MAST, Culham, UK** - A mega-ampere class ST with many complementary features to NSTX. Large vacuum vessel, no stabilizing plates, excellent set of profile and boundary diagnostics. NBI and EBW heating. Confinement, H-mode, and boundary physics. Innovative start-up utilizing internal PFs.

**GLOBUS-M, Ioffe Physico, RF** - Medium size ST with  $R/a = 1.5$ ,  $I_p \sim 300$  kA. ICRF, NBI, Plasma Jet.

**TST-2, Tokyo University, Japan** - Small size ST with  $I_p \sim 140$  kA. HHFW and EBW physics.

**TS3 and 4, Tokyo University, Japan** - Small size STs. Spheromak merging to obtain high beta STs with internal coils. Short pulse with  $I_p < 300$  kA.

**LATE, Kyoto University, Japan** - Small size ST. ECH/EBW for start-up without OH with  $I_p \sim 10$  kA.

There are a number other small STs in China, Brazil, and Japan.

- **STs under construction:**

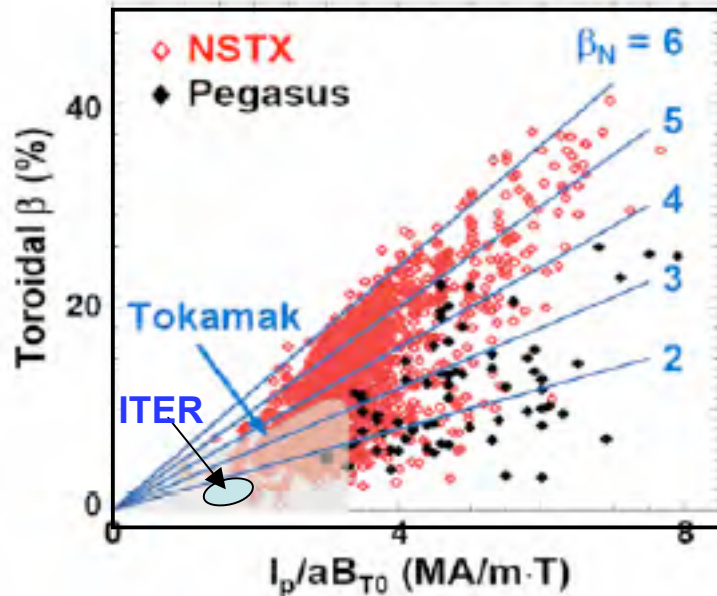
**QUEST, Kyushu University, Japan** - Follow-on device of TRIAM. Medium size, long-pulse, non-inductive operations. First stage to start in  $\sim 2008$  with  $I_p < 100$  kA.

**UTST, Tokyo University, Japan** - Follow-on device to TS3/4 and TST-2. Double null formation with more reactor relevant external PF coils. Aim to achieve very high beta with merging. First Plasma in 2007.

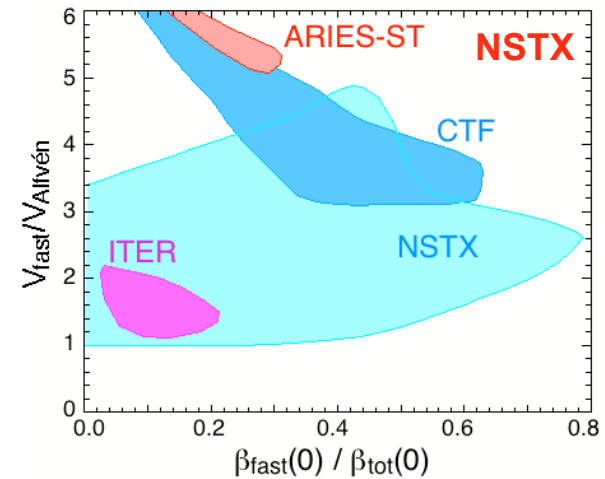
# ST Offers Access to Wide Tokamak Plasma Regimes



Wide range of  $\beta_T$  up to  $\sim 40\%$ .

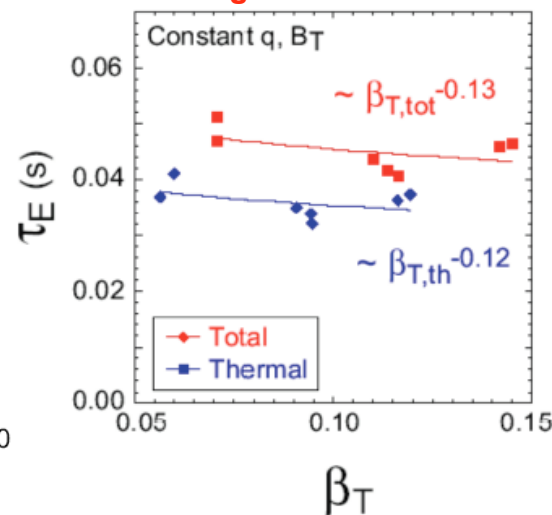
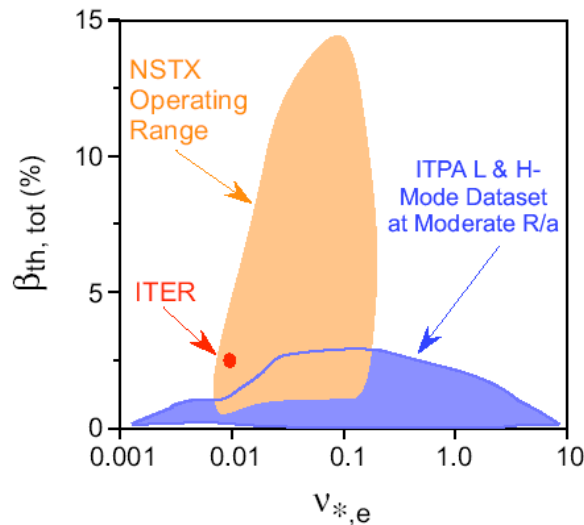


Unique Energetic Particle Physics

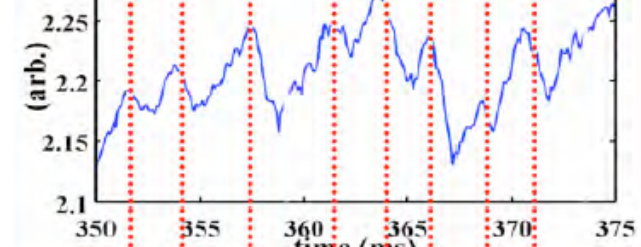


$\beta$  Confinement Scaling, Electron Transport

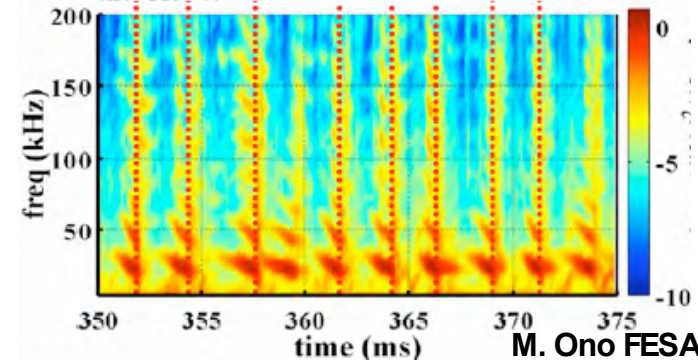
Weak beta degradation observed



Multi-mode driven losses



TAE/EPM

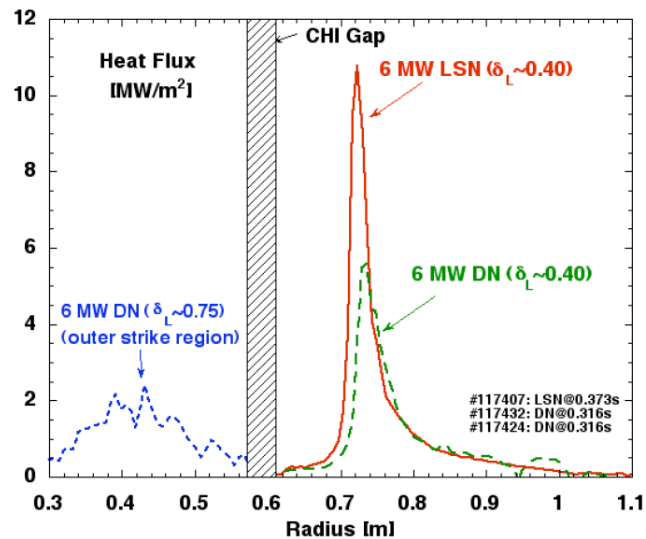


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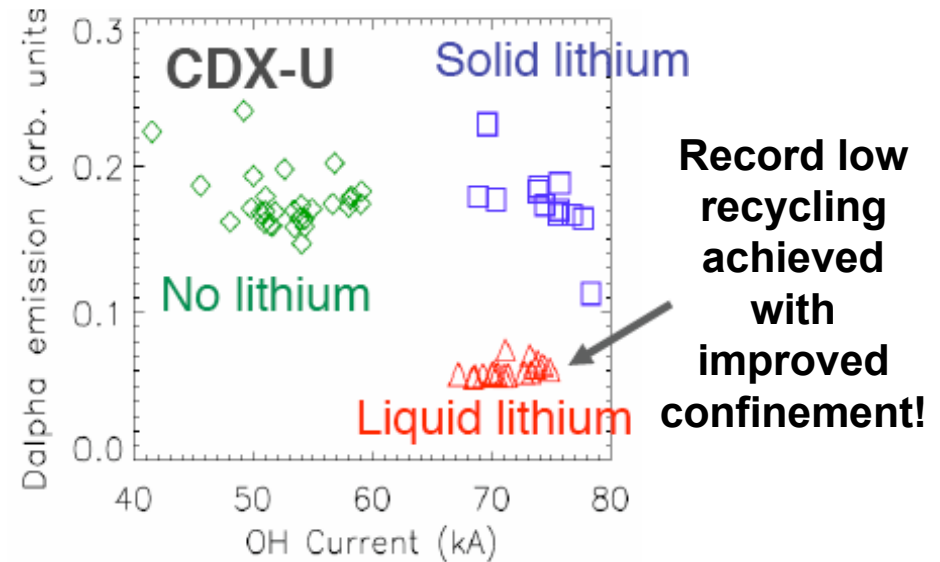
# Investigating Innovative Solutions for NHTX/CTF and DEMO



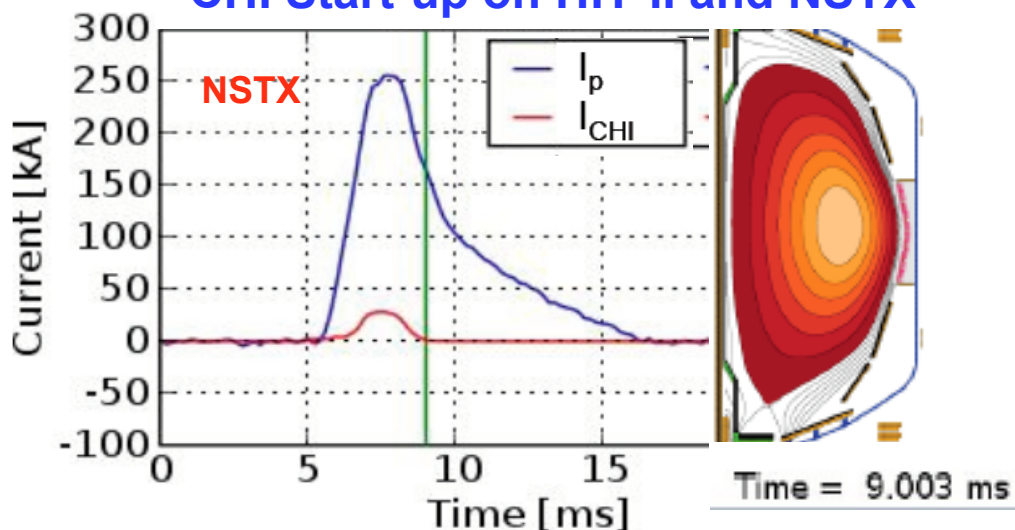
## NSTX with ITER-level heat flux



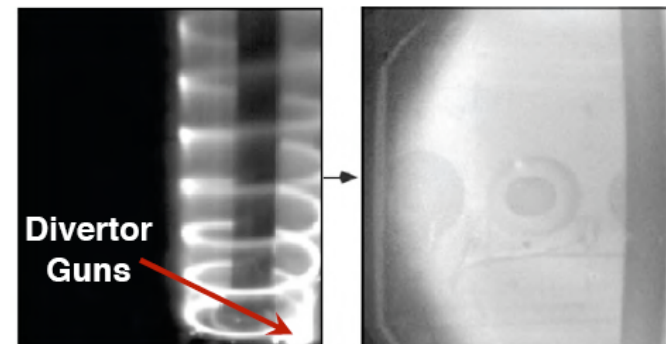
## Lithium on CDX-U/LTX and NSTX



## CHI Start-up on HIT-II and NSTX



## PEGASUS Gun Start-up



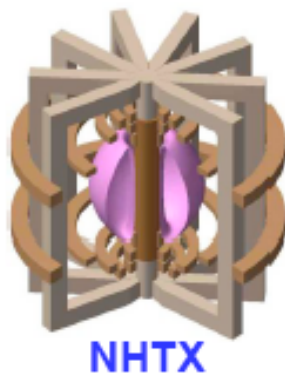
$I_p \sim 30$  kA achieved



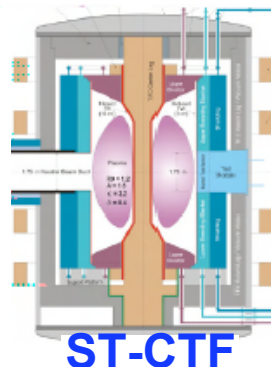
# Next Steps in ST Development Path



**Present ST program contributes physics basis for the crucial next step ST devices**



**NHTX (PE-ST) at 3 - 4 MA range provides cost effective physics / technology basis in support for CTF and DEMO: Demonstrate stable continuous high-performance operation with very high heat flux and acceptable hydrogen isotope retention.**



**Compact ST-CTF at ~ 10 MA range provide a compact nuclear component test facility to support DEMO: Provide ~ 6 Mwyr/m<sup>2</sup> neutron over ~ 10 m<sup>2</sup> with acceptable level of tritium consumption**

# Broad Contributions of ST Research Program

